

# MCNPX – New Features Demonstrated

G. W. McKinney

J. W. Durkee, J. S. Hendricks, M. R. James, D. B. Pelowitz, L. S.  
Waters

*Los Alamos National Laboratory  
Los Alamos, NM*

F. X. Gallmeier

*Oak Ridge National Laboratory  
Oak Ridge, TN*

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# Contributors

## **Code Development Team**

Joe W. Durkee, Harry W. Egdorf, Franz Gallmeier, John S. Hendricks,  
Gregg W. McKinney, Teresa L. Roberts, Laurie S. Waters, Joshua P.  
Finch

## **Library Development Team**

Mark B. Chadwick, Stephanie C. Frankle, Gerald M. Hale, Robert C.  
Little, Robert MacFarlane, Morgan C. White, Phillip G. Young

## **Physics Development Team**

David G. Madland, Stepan Mashnik, Richard E. Prael, Arnold J. Sierk,  
Jean-Christophe David, Alfredo Ferrari

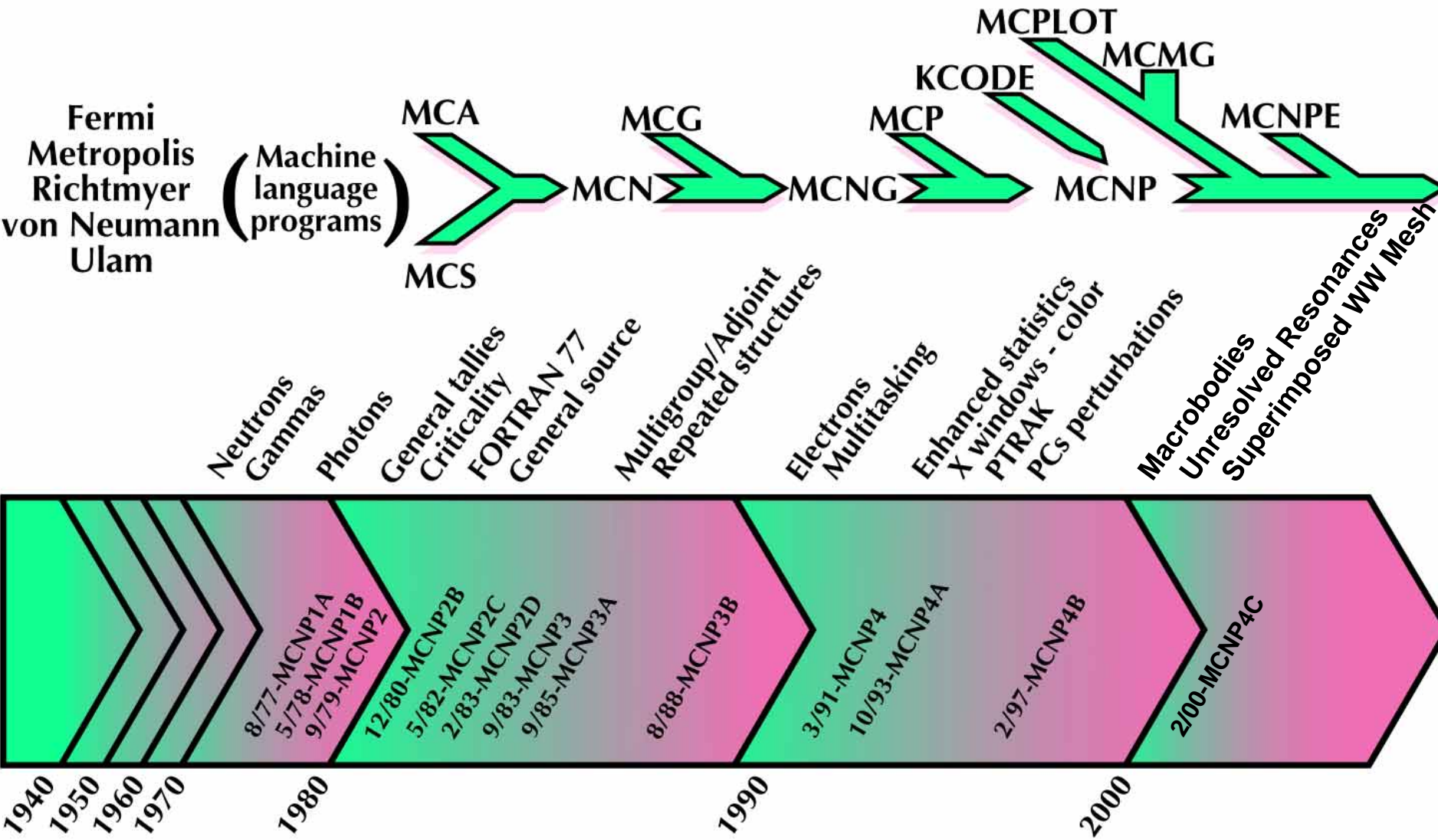
# Outline

- **Overview**
- **Development History**
- **User Base**
- **New 2.5.0 Features**
- **New 2.6.x Features**
- **Future Development**

# Overview

- **Monte Carlo radiation transport code**
  - Extends MCNP 4C to virtually all particles and energies
  - 34 particle types (n,p,e, 5 Leptons, 11 Baryons, 11 Mesons, 4 LI)
  - Continuous energy (roughly 0-100 GeV)
  - Data libraries below  $\sim 150$  MeV (n,p,e,h) and models otherwise
- **General 3-D geometry**
  - 1<sup>st</sup> & 2<sup>nd</sup> degree surfaces, tori, 10 macrobodies, lattices
- **General sources and tallies**
  - Interdependent source variables, 7 tally types, many modifiers
- **Supported on virtually all computer platforms**
  - Unix, Linux, Windows, OS X (parallel with PVM or MPI)

# HISTORY OF MCNP DEVELOPMENT



# Development History

- **MCNP & LAHET Merger Project**                      **1995**
- **Version 2.1.5**    **November 14, 1999**
  - HISTP/HTAPE3X, Mesh & radiography tallies, CEM
- **Version 2.3.0**    **April 27, 2002**
  - Proton libraries
- **Version 2.4.0**    **August 1, 2002**
  - Update to MCNP 4C, Fortran 90, Windows PC support
- **Version 2.5.0**    **March 21, 2005**
  - Twenty-eight features

# User Base

- **Over 2000 users world wide**
  - Provide 6-8 workshops per year (4-6 US, ~2 international)
  - 150 workshop participants per year
  - Access to RSICC/NEA released versions only
    - <http://rsicc.ornl.gov>
  - Limited access to MCNPX web site
    - <http://mcnpx.lanl.gov> (some documentation)
- **Over 1500 registered Beta Testers**
  - Full access to MCNPX web site
  - Access to intermediate versions
  - Increased user support

<b>Application</b>	<b># Groups</b>	<b>Percent</b>
Medical (BNCT, proton therapy, etc.)	50	15
Spacecraft, Cosmic Rays, SEE, propulsion	42	13
Detectors, experiments, Threat Reduction	39	12
ATW, ADS, Energy Amplifiers	37	11
Fuel cycles, beginning to end, including storage	32	10
Accelerator Shielding and Health Physics	28	8
Theoretical Physics	23	7
Neutron Production for Scattering	21	6
Isotope Production	14	4
Radiography	12	4
MCNPX/MCNP code development	11	3
Materials studies (IFMIF)	6	2
Radioactive Ion Beams	5	2
Irradiation Facilities	4	1
Neutrino Targets	4	1
Light Sources, electron machines	3	1



# New 2.5.0 Features

- **User-interface enhancements (15)**
  - 5 new source options
  - 4 new tally options
  - 3 new graphics options
  - 3 other miscellaneous improvements
- **Physics enhancements (9)**
  - 4 new model physics features
  - 2 new neutron physics features
  - 3 new photon physics features
- **Infrastructure enhancements (4)**

# User-Interface Enhancements

- **Five new source options**
  - Positron sources
  - Spontaneous fission sources
  - Multiple source particles
  - Default VEC for cylindrical sources
  - Extension of the TR keyword

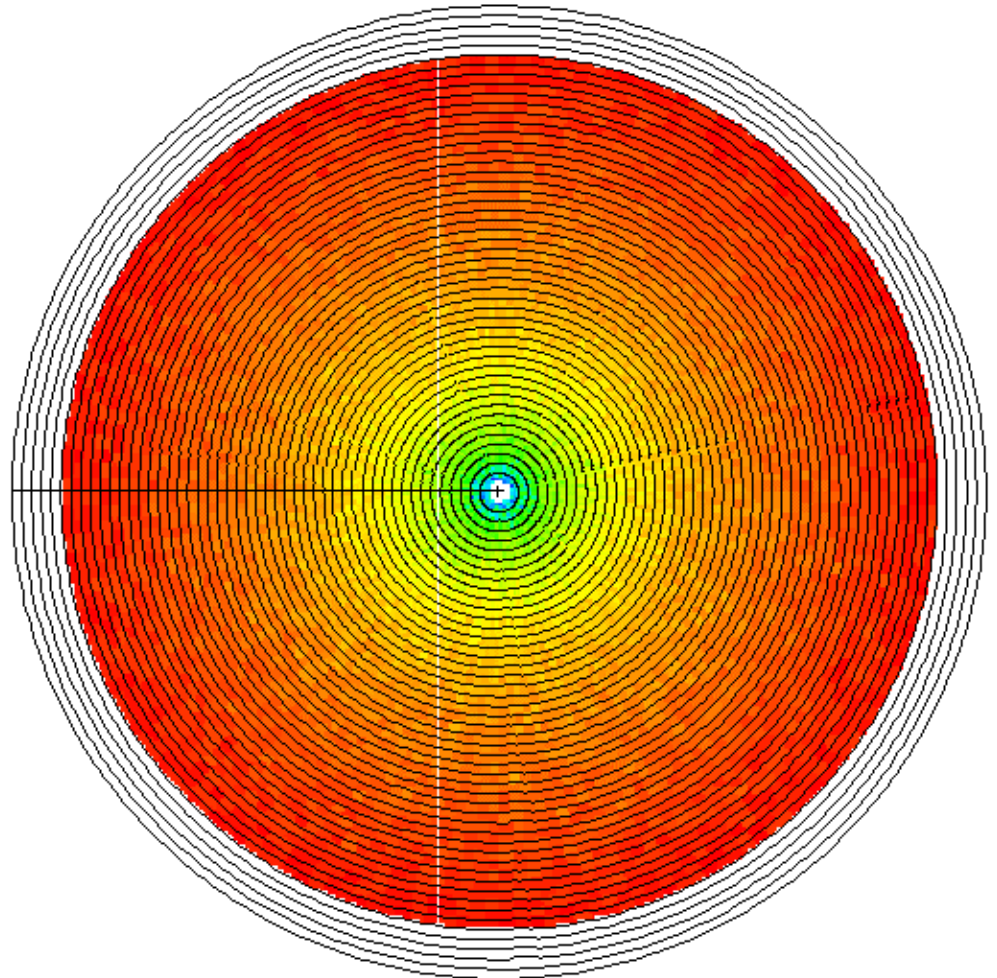
# Positron Sources

## 1.0 MeV Positron Source on a Disk

```
1 0 -1 imp:n=1
2 0 1 imp:n=0

1 SPH 0 0 0 100

mode e
sdef par=-e erg=1.0
      pos=0 0 0 rad=d1 axs=0 0 1 ext=0
sil 0 90
spl -21 1
nps 1000000
tmesh
  cmesh2 e
  cora2 2 48i 100
  corb2 -1 1
  corc2 2 178i 360
endmd
print
```



# Spontaneous Fission Sources

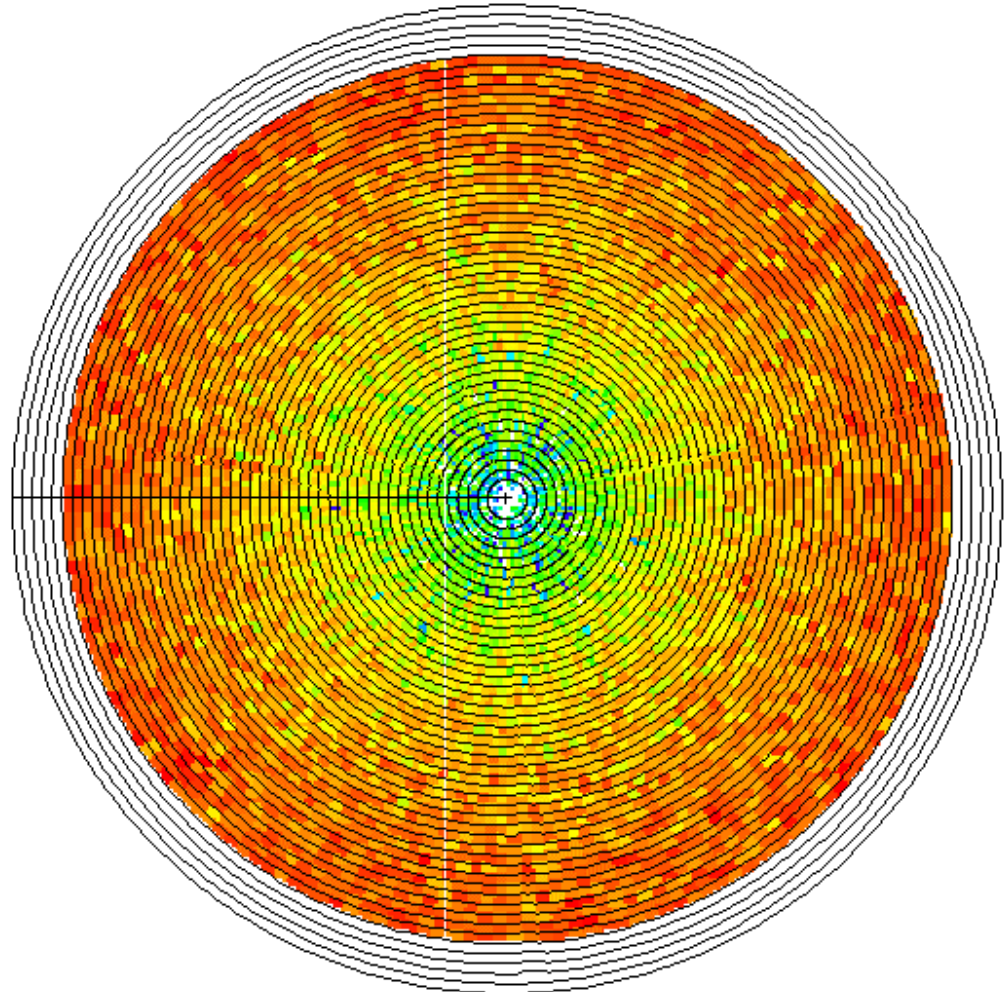
Pu-239 Spontaneous Fission Source in H2O

```
1 1 -1.0 -1 imp:n=1
2 0 1 imp:n=0

1 SPH 0 0 0 100

sdef par=sf
      pos=0 0 0 rad=d1 axs=0 0 1 ext=0
sil 0 90
spl -21 1
m1 1001 2 8016 1 94239 1.e-4
nps 100000

tmesh
  cmesh2 n
  cora2 2 48i 100
  corb2 -1 1
  corc2 2 178i 360
endmd
print
```



# Multiple Source Particles / TR Extension

## Distribution for PAR and TR Keywords

```
1 0 -1 imp:n=1
2 0 1 imp:n=0
```

```
1 SPH 0 0 0 100
```

```
mode n p
```

```
sdef par=d1 erg=fpar=d2 tr=fpar=d3
      x=d4 y=d5 z=0 cell=1
```

```
si1 L n p
```

```
sp1 1 1
```

```
ds2 L 1.0 2.0
```

```
ds3 L 1 2
```

```
si4 -50 50
```

```
sp4 0 1
```

```
si5 -50 50
```

```
sp5 0 1
```

```
tr1 -50 50 0
```

```
tr2 50 -50 0
```

```
nps 10000
```

```
tmesh
```

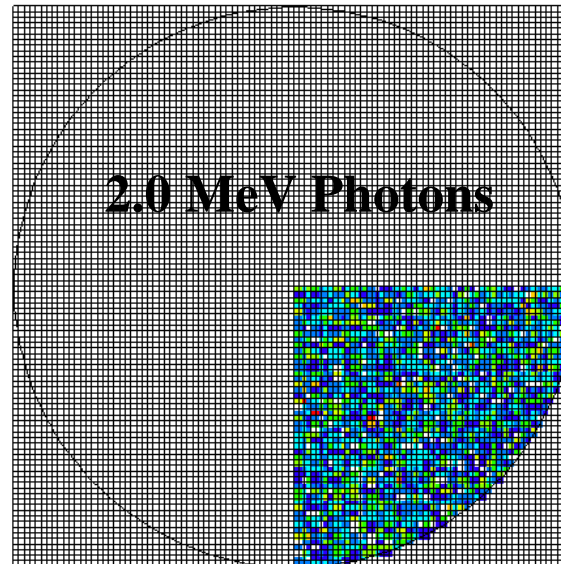
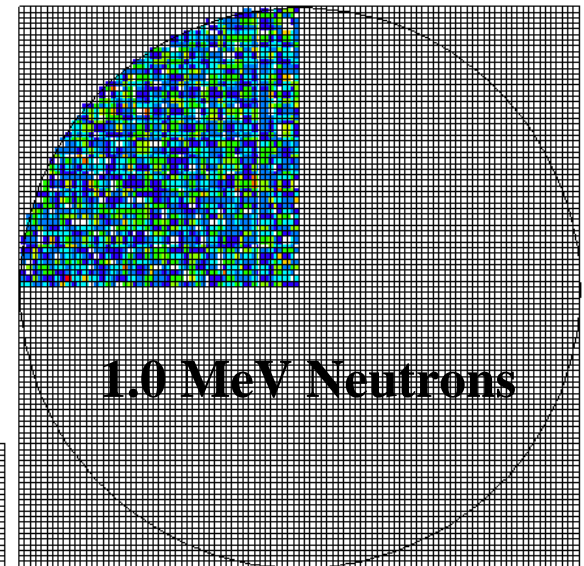
```
rmesh2 n p
```

```
cora2 -100 99i 100
```

```
corb2 -100 99i 100
```

```
corc2 -1 1
```

```
endmd
```



# Default VEC for Cylindrical Sources

## Cylindrical Source with Default VEC

```
1 0 -1 2 imp:n=1  
2 0 -2:1 imp:n=0
```

```
1 SPH 0 0 0 100  
2 SPH 0 0 0 1
```

```
sdef pos=0 0 0 rad=90 axs=0 0 1 ext=0  
dir=1 nrm=-1
```

```
nps 100
```

```
tmesh
```

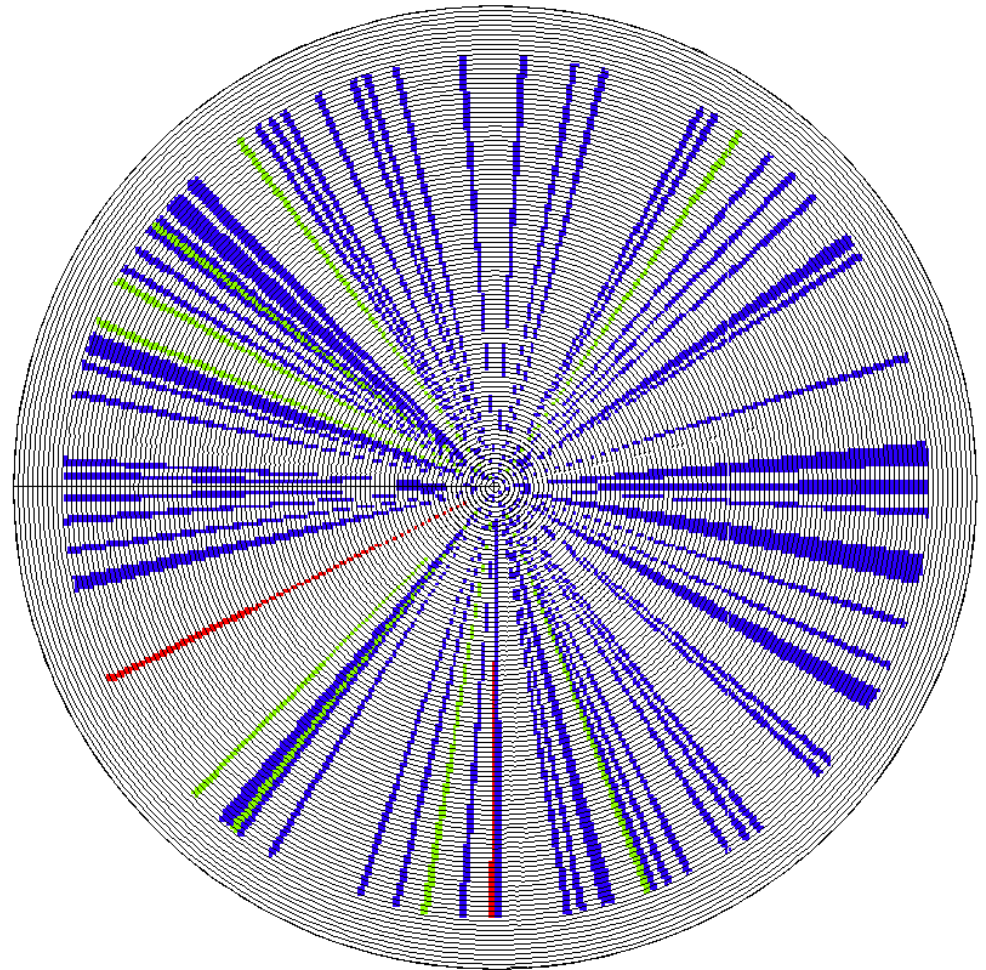
```
cmesh1:n traks
```

```
cora1 1 98i 100
```

```
corb1 -1 1
```

```
corc1 1 358i 360
```

```
endmd
```



# User-Interface Enhancements

- **Four new tally options**
  - Lattice tally speedup
  - Anticoincidence pulse-height tally
  - Coincidence capture pulse-height tally
  - Residual nuclei pulse-height tally

# Lattice Tally Speedup

5 MeV Photons into a 100x100x100 Water Phantom

```
1 1 -1.0 -1 u=1 imp:p=1
2 1 -1.0 1 u=1 imp:p=1
3 0 -2 u=2 fill=1 imp:p=1
4 0 -3 u=3 imp:p=1
    lat=1 fill=-50:49 -50:49 -50:49 2 999999R
5 0 -4 fill=3 imp:p=1
6 0 4 imp:p=0

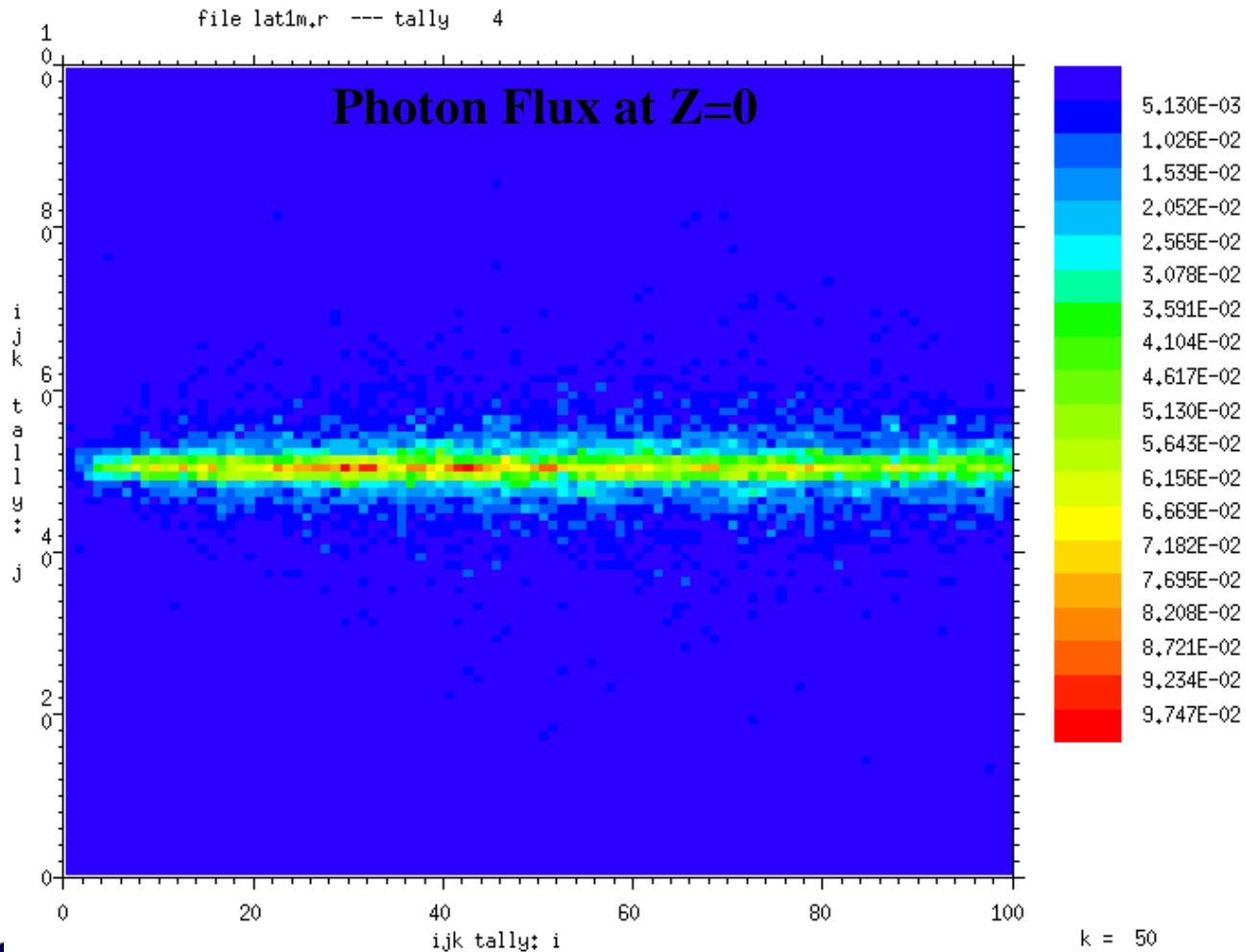
1 sph 0 0 0 .05
2 sph 0 0 0 10
3 rpp -.1 .1 -.1 .1 -.1 .1
4 rpp -10.099 9.899 -10.099 9.899 -10.099 9.899

mode p
m1 1000 2 8000 1
sdef erg=5 pos=-10.098 0 0 vec=1 0 0 dir=1
nps 100000
f4:p (1<3<4[-50:49 -50:49 -50:49]<5)
sd4 1
fm4 1.0
de4 1e-11 100
df4 1 1
talnp
```

	Startup Time (m)	Tracking Time (part./m)	Speedup
MCNPX 2.5.0	.02	23,300	17,260
MCNPX 2.4.0	230	1.35	1
MCNP5 1.20	226	1.47	~1
MCNP5 1.30	220	Crashed	--



# Lattice Tally Speedup



# Anticoincidence Pulse-Height Tally

Anticoincidence PHT 1 MeV Photons => Plastic/BGO

```
1 1 -7.130 -1      imp:p=1
2 2 -1.032  1 -2  3 imp:p=1
3 0          1 -2 -3 imp:p=1
4 0          2      imp:p=0
```

```
1 SPH 0 0 0 5.0
2 SPH 0 0 0 6.0
3 RCC -7 0 0 4 0 0 3.0
```

mode p e

sdef sur=2 nrm=-1 par=p erg=1.0

nps 100000

m1 83000 -0.671 32000 -0.175 8000 -0.154

m2 6000 -0.9153 1000 -0.0847

f26:e 2 \$ Plastic energy dep.

ft26 GEB 0 0.1098 0

sd26 1

f36:e 1 \$ BGO energy dep.

ft36 GEB 0 0.1098 0

sd36 1

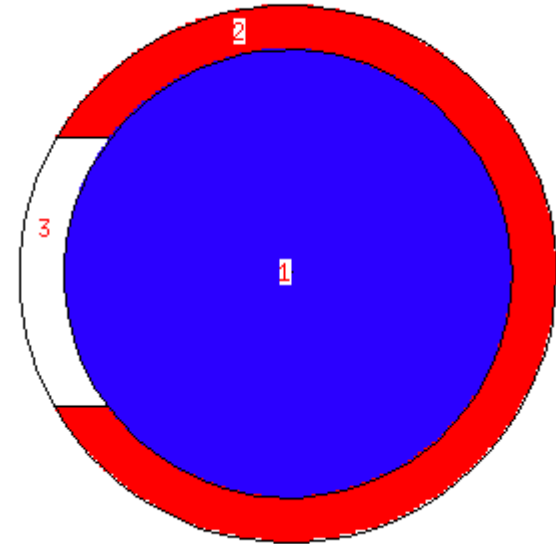
f18:e 1 \$ Plastic/BGO PHT

e18 0. 1.0

fu18 0. 99i 1.0


ft18 ph1 1 26 1 1 36 1

fq18 u e

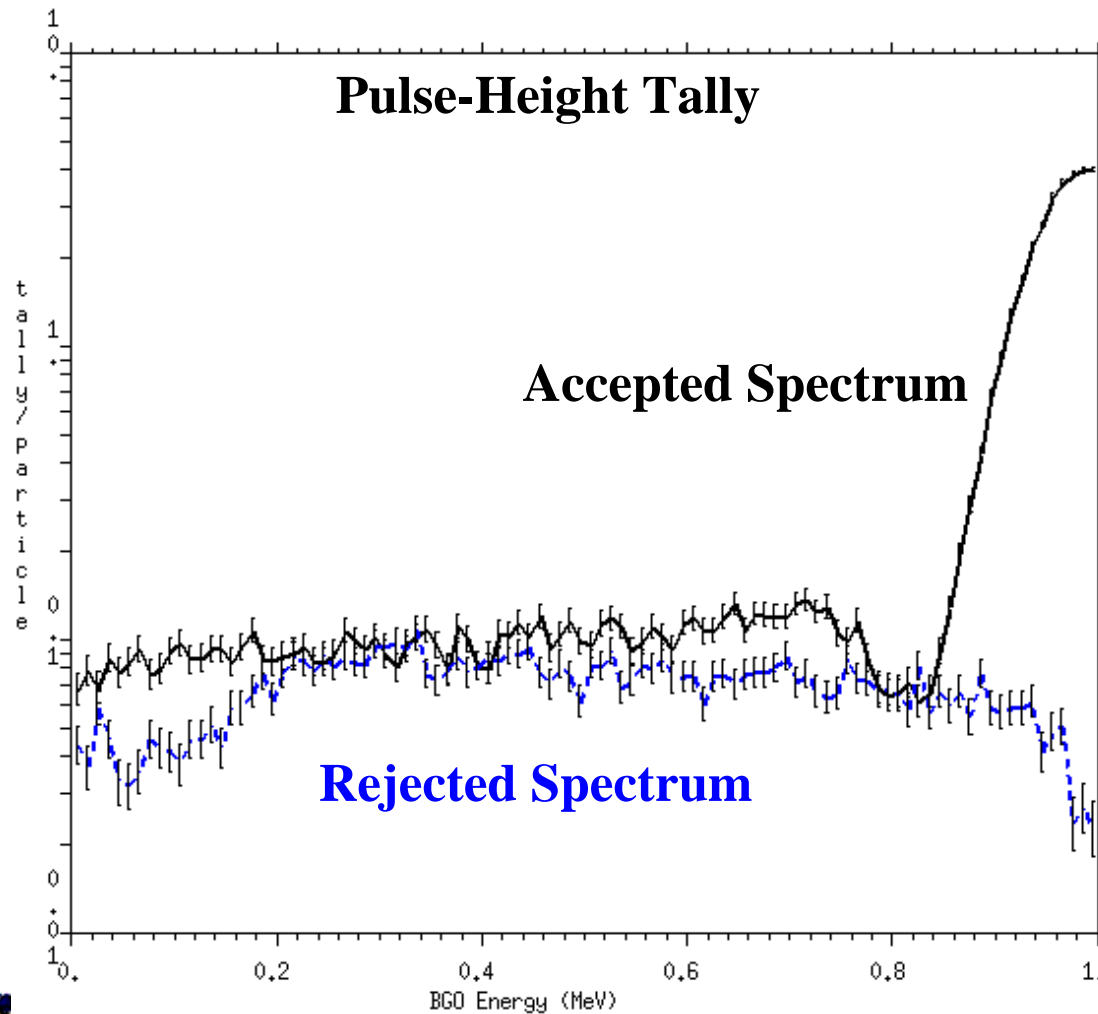


# Anticoincidence Pulse-Height Tally

```
ltally 18      nps = 100000
+
                ACS/BGO Pulse Height Response - all particles
tally type 8   pulse height distribution.           units  number
particle(s):  photon      electron
this tally is modified by  ft  phl
```

cell 1	energy:	0.0000E+00	1.0000E+00	total	← Plastic		
user bin							
0.0000E+00	1.30000E-04	0.2773	8.84300E-02	0.0102	8.85600E-02	0.0101	<b>B</b> <b>G</b> <b>O</b> 
1.0000E-02	6.80000E-04	0.1212	4.40000E-04	0.1507	1.12000E-03	0.0944	
2.0000E-02	7.90000E-04	0.1125	3.70000E-04	0.1644	1.16000E-03	0.0928	
3.0000E-02	6.90000E-04	0.1203	5.90000E-04	0.1302	1.28000E-03	0.0883	
4.0000E-02	8.70000E-04	0.1072	4.60000E-04	0.1474	1.33000E-03	0.0867	
5.0000E-02	7.80000E-04	0.1132	3.30000E-04	0.1740	1.11000E-03	0.0949	
6.0000E-02	8.40000E-04	0.1091	3.20000E-04	0.1767	1.16000E-03	0.0928	
7.0000E-02	9.30000E-04	0.1036	3.60000E-04	0.1666	1.29000E-03	0.0880	
8.0000E-02	7.60000E-04	0.1147	4.60000E-04	0.1474	1.22000E-03	0.0905	
9.0000E-02	8.00000E-04	0.1118	4.30000E-04	0.1525	1.23000E-03	0.0901	
1.0000E-01	9.20000E-04	0.1042	4.20000E-04	0.1543	1.34000E-03	0.0863	
1.1000E-01	9.80000E-04	0.1010	3.80000E-04	0.1622	1.36000E-03	0.0857	
1.2000E-01	8.60000E-04	0.1078	4.60000E-04	0.1474	1.32000E-03	0.0870	
1.3000E-01	8.70000E-04	0.1072	4.60000E-04	0.1474	1.33000E-03	0.0867	
1.4000E-01	9.30000E-04	0.1036	5.20000E-04	0.1386	1.45000E-03	0.0830	
1.5000E-01	9.30000E-04	0.1036	4.30000E-04	0.1525	1.36000E-03	0.0857	
1.6000E-01	8.40000E-04	0.1091	5.90000E-04	0.1302	1.43000E-03	0.0836	
1.7000E-01	9.50000E-04	0.1025	5.90000E-04	0.1302	1.54000E-03	0.0805	
1.8000E-01	1.08000E-03	0.0962	6.70000E-04	0.1221	1.75000E-03	0.0755	

# Anticoincidence Pulse-Height Tally



# Coincidence Capture Pulse-Height Tally

## Coincidence Capture Tally Pu-239 in H2O/B-10

```
1 1 -1.0 -1 imp:n=1
2 2 -1.0 1 -2 imp:n=1
3 0 2 imp:n=0
```

```
1 SPH 0 0 0 10
2 SPH 0 0 0 20
```

```
sdef par=sf pos=0 0 0 rad=d1
sil 0 10
spl -21 2
m1 1001 2 8016 1 94239 1.e-4
m2 5010 1
nps 100000
f8:n 2
ft8 CAP -8 -8 5010
t8 1e2 1e5 1e8 $ 1us, 1ms, 1s
print
```

# Coincidence Capture Pulse-Height Tally

1 neutron captures, moments and multiplicity distributions. tally 8

print table 118

weight normalization by source fission neutrons = 215705

cell: 2

time bin: 1.0000E+02

neutron captures on 10b

	histories	captures by number	captures by weight	multiplicity by number	fractions by weight	error
captures = 0	38079	0	0.00000E+00	3.80790E-01	1.76533E-01	0.0040
captures = 1	39130	39130	1.81405E-01	3.91300E-01	1.81405E-01	0.0039
captures = 2	17769	35538	1.64753E-01	1.77690E-01	8.23764E-02	0.0068
captures = 3	4429	13287	6.15980E-02	4.42900E-02	2.05327E-02	0.0147
captures = 4	555	2220	1.02918E-02	5.55000E-03	2.57296E-03	0.0423
captures = 5	38	190	8.80833E-04	3.80000E-04	1.76167E-04	0.1622
total	100000	90365	4.18929E-01	1.00000E+00	4.63596E-01	0.0031

factorial moments	by number		by weight	
10b	9.03650E-01	0.0031	4.18929E-01	0.0031
10b(10b-1)/2!	3.47660E-01	0.0076	1.61174E-01	0.0076
10b(10b-1)(10b-2)/3!	7.02900E-02	0.0183	3.25862E-02	0.0183
10b(10b-1)....(10b-3)/4!	7.45000E-03	0.0520	3.45379E-03	0.0520
10b(10b-1)....(10b-4)/5!	3.80000E-04	0.1622	1.76167E-04	0.1622

# Coincidence Capture Pulse-Height Tally

cell: 2

time bin: 1.0000E+05

neutron captures on 10b

	histories	captures by number	captures by weight	multiplicity by number	fractions by weight	error
captures = 0	64457	0	0.00000E+00	6.44570E-01	2.98820E-01	0.0023
captures = 1	27555	27555	1.27744E-01	2.75550E-01	1.27744E-01	0.0051
captures = 2	6212	12424	5.75972E-02	6.21200E-02	2.87986E-02	0.0123
captures = 3	1263	3789	1.75657E-02	1.26300E-02	5.85522E-03	0.0280
captures = 4	362	1448	6.71287E-03	3.62000E-03	1.67822E-03	0.0525
captures = 5	86	430	1.99346E-03	8.60000E-04	3.98693E-04	0.1078
captures = 6	46	276	1.27953E-03	4.60000E-04	2.13254E-04	0.1474
captures = 7	12	84	3.89421E-04	1.20000E-04	5.56315E-05	0.2887
captures > 7	7	58	2.68886E-04	7.00000E-05	3.24517E-05	0.3780
total	100000	46064	2.13551E-01	1.00000E+00	4.63596E-01	0.0050

factorial moments

by number

by weight

10b	4.60640E-01	0.0050	2.13551E-01	0.0050
10b(10b-1)/2!	1.41880E-01	0.0172	6.57750E-02	0.0172
10b(10b-1)(10b-2)/3!	5.36700E-02	0.0533	2.48812E-02	0.0533
10b(10b-1) .... (10b-3)/4!	2.53200E-02	0.1255	1.17383E-02	0.1255
10b(10b-1) .... (10b-4)/5!	1.20200E-02	0.2487	5.57243E-03	0.2487
10b(10b-1) .... (10b-5)/6!	5.08000E-03	0.4377	2.35507E-03	0.4377
10b(10b-1) .... (10b-6)/7!	1.80000E-03	0.6758	8.34473E-04	0.6758
10b(10b-1) .... (10b-7)/8!	5.10000E-04	0.8837	2.36434E-04	0.8837

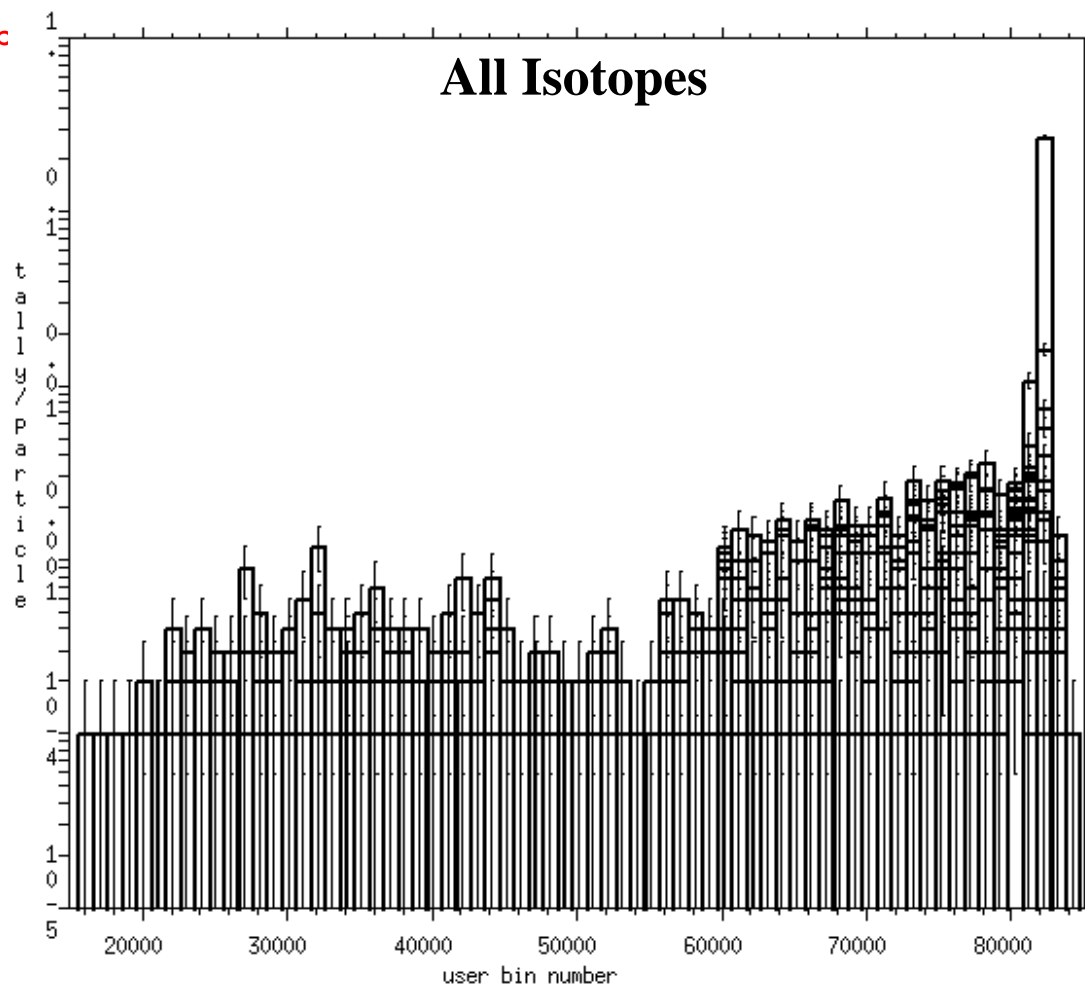
# Residual Nuclei Pulse-Height Tally

Residuals for 1.2 GeV Protons => Pb

```
1 1 -11. -1 imp:h 1
2 0      1 imp:h 0

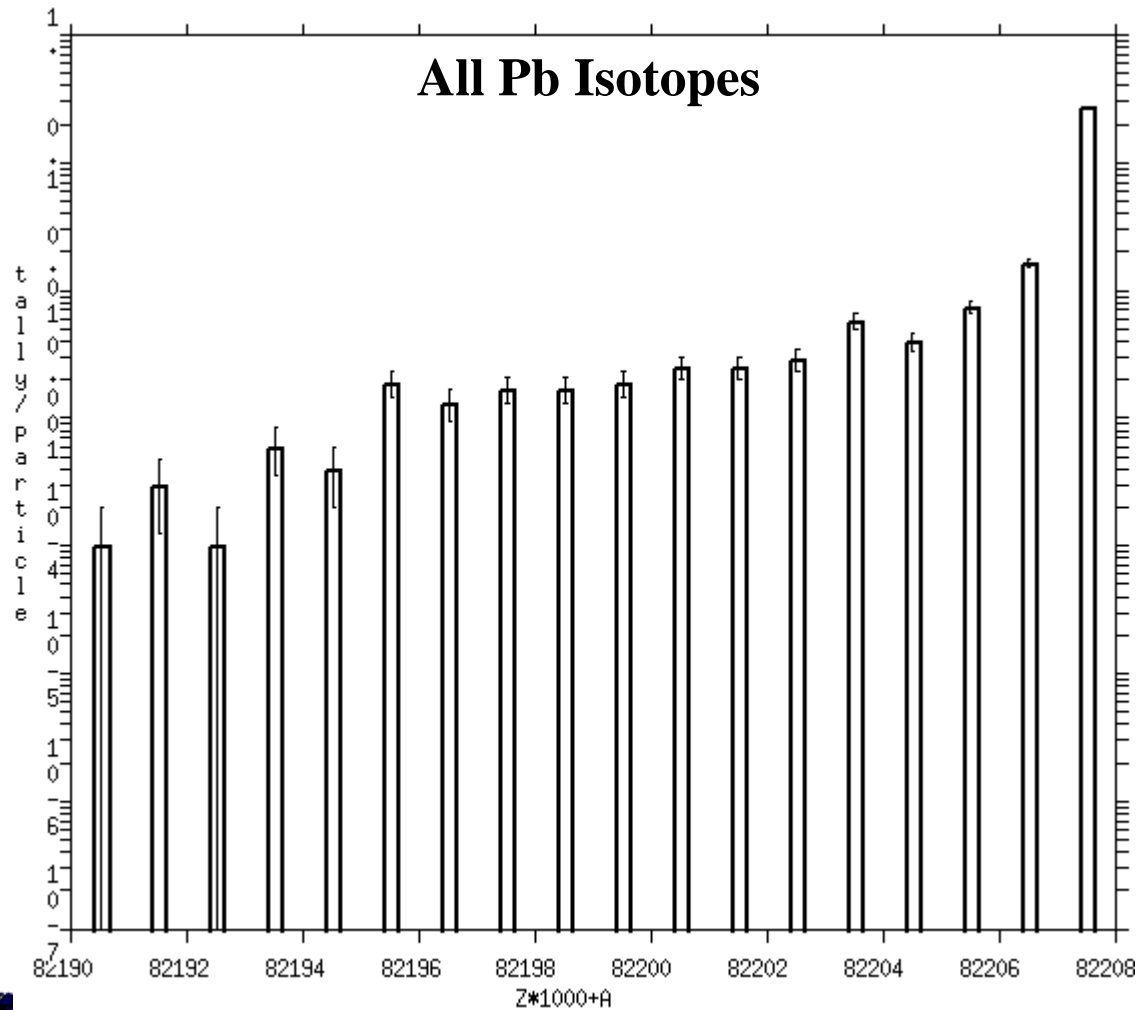
1 so .01

mode h n
sdef par h erg=1200
      vec 0 0 1 dir 1
m1 82208 1
phys:h 1300 j 0
phys:n 1300 3j 0
nps 10000
f8:h 1
ft8 RES 1 99
lca 7j -2 0
```





# Residual Nuclei Pulse-Height Tally



# User-Interface Enhancements – 2.5.0

- **Three new graphics options**
  - Lattice index labeling
  - WWG superimposed mesh plots
  - Color contour and mesh tally plots

# Lattice Index Labeling

5 MeV Photons => 100x100x100 Water Phantom

```
1 1 -1.0 -1 u=1 imp:p=1
2 1 -1.0 1 u=1 imp:p=1
3 0 -2 u=2 fill=1 imp:p=1
4 0 -3 u=3 imp:p=1
    lat=1 fill=-50:49 -50:49 -50:49 2 999999R
5 0 -4 fill=3 imp:p=1
6 0 4 imp:p=0

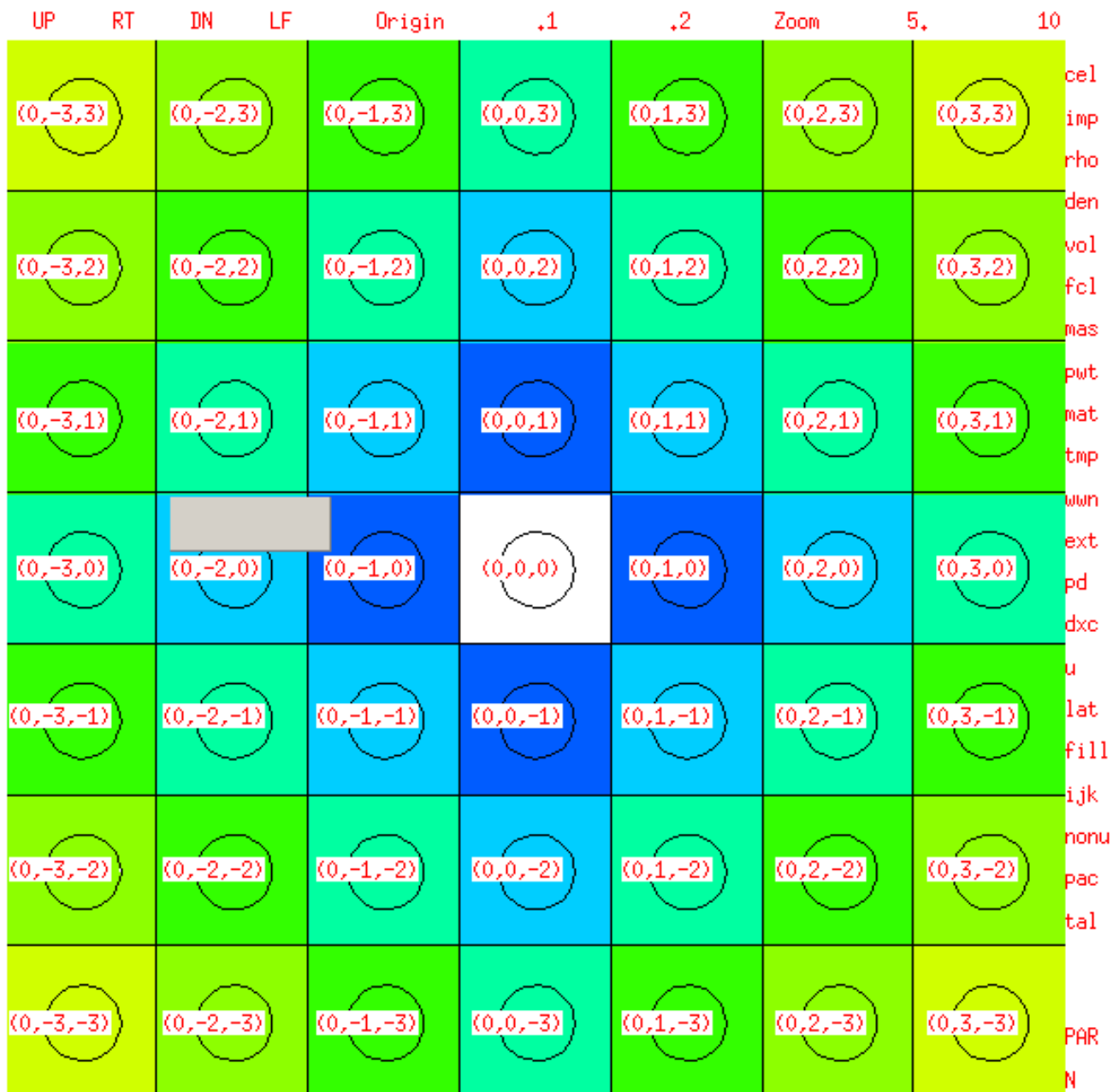
1 sph 0 0 0 .05
2 sph 0 0 0 10
3 rpp -.1 .1 -.1 .1 -.1 .1
4 rpp -10.099 9.899 -10.099 9.899 -10.099 9.899

mode p
m1 1000 2 8000 1
sdef erg=5 pos=-10.098 0 0 vec=1 0 0 dir=1
nps 100000
f4:p (1<3<4[-50:49 -50:49 -50:49]<5)
sd4 1
fm4 1.0
de4 1e-11 100
df4 1 1
talnp
```

04/12/05 16:44:08  
100x100x100 water phantom

probid = 04/12/05 16:42:17  
basis: YZ  
( 0.000000, 1.000000, 0.000000)  
( 0.000000, 0.000000, 1.000000)  
origin:  
( 0.00, 0.00, 0.00)  
extent = ( 0.70, 0.70)  
cell labels are  
lattice indices ijk

Edit ijk (0,0,0)  
Cell 1  
xyz = 0.00, 0.00, 0.00  
CURSOR SCALES 0 CellLine  
PostScript ROTATE  
COLOR ijk LEVEL  
XY YZ ZX  
LABEL off ijk  
MBODY on



[Click here](#) or [picture](#) or [menu](#)

Redraw

Plot>

End

# WWG Superimposed Mesh Plots

Cylindrical WW Mesh-3 MeV Photons => H2O

```
1 1 1.0 -1 imp:p 1
2 0 1 imp:p 0
```

```
1 rcc 0 0 0 0 10 0 5
```

```
mode p
```

```
sdef sur=1.3 vec=0 1 0 dir=1 erg=3
```

```
m1 1001 2 8016 1
```

```
nps 1000000
```

```
f1:p 1.2
```

```
wwg 1 0
```

```
mesh geom=cyl origin=0 -1 0 ref=0 .1 0
```

```
axs=0 1 0 vec=1 0 0
```

```
imesh 6 iints 7
```

```
jmesh 12 jint 7
```

```
kmesh 1 kints 3
```

Cylindrical WW Mesh-3 MeV Photons => H2O

```
1 1 1.0 -1 imp:p 1
2 0 1 imp:p 0
```

```
1 rcc 0 0 0 0 10 0 5
```

```
mode p
```

```
sdef sur=1.3 vec=0 1 0 dir=1 erg=3
```

```
m1 1001 2 8016 1
```

```
nps 1000000
```

```
f1:p 1.2
```

```
wwg 1 0
```

```
mesh geom=cyl origin=0 -1 0 ref=0 .1 0
```

```
axs=0 1 0 vec=1 0 0
```

```
imesh 6 iints 14
```

```
jmesh 12 jint 14
```

```
kmesh 1 kints 6
```

```
wwp:p 4j -1
```

04/13/05 14:32:42  
Demonstration of WWG Plot

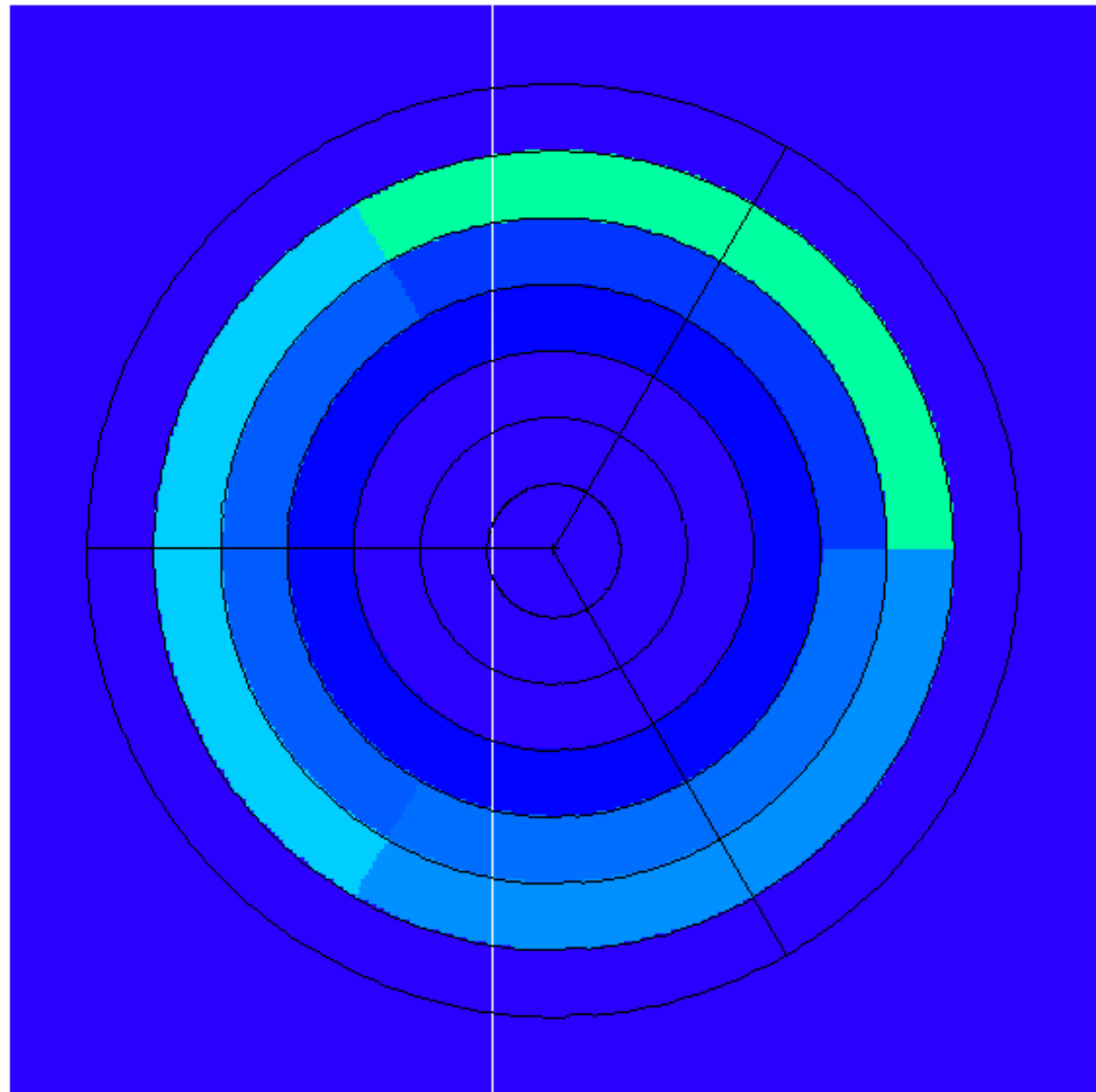
UP RT DN LF Origin .1 .2 Zoom 5. 10

```
probid = 04/13/05 14:29:41
basis: XZ
( 1.000000, 0.000000, 0.000000)
( 0.000000, 0.000000, 1.000000)
origin:
( 0.01, 3.00, 0.01)
extent = ( 7.00, 7.00)
```

\*

```
Edit cel 1
          Cell 1
xyz = 0.01, 3.00, 0.01
CURSOR SCALES 0 WW MESH
PostScript ROTATE
COLOR wwn1:p
XY YZ ZX
LABEL off off
MBODY on
```

[Click here](#) or [picture](#) or [menu](#)



cel  
imp  
rho  
den  
vol  
fcl  
mas  
pwt  
mat  
tmp  
wwn  
ext  
pd  
dxc  
u  
lat  
fill  
ijk  
nonu  
pac  
tal  
PAR  
N

Redraw Plot> End

04/13/05 14:37:01  
Demonstration of MWG Plot

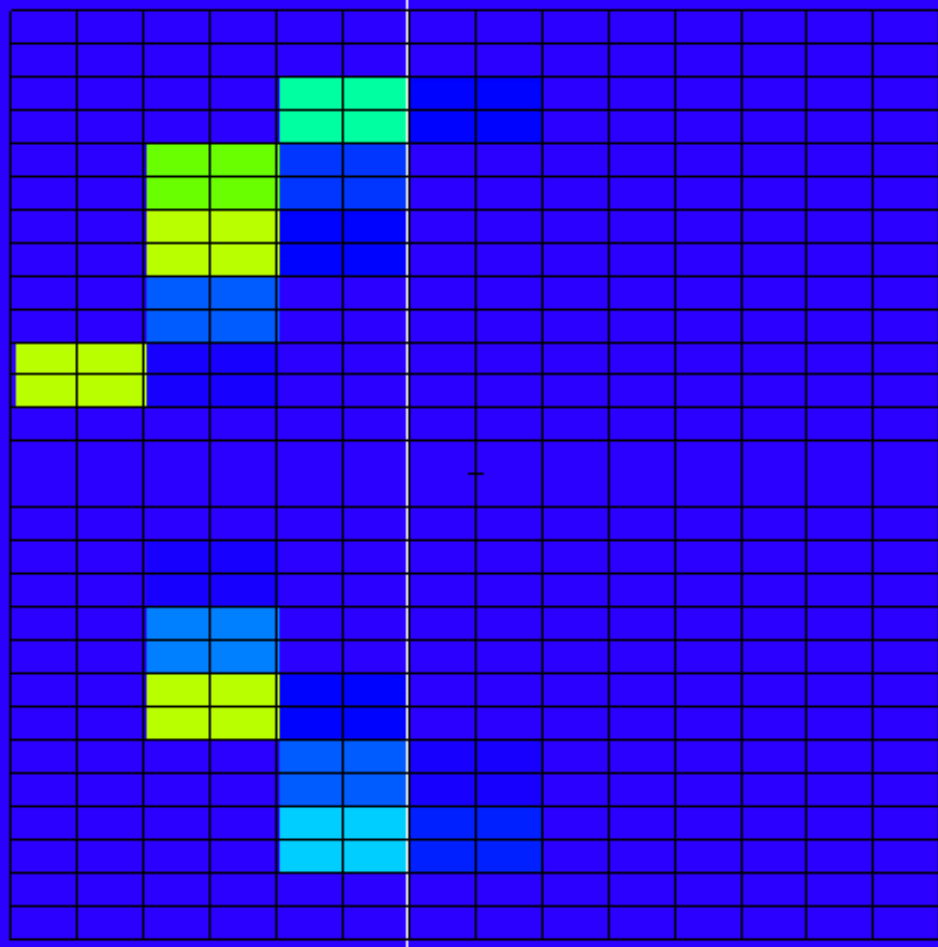
UP RT DN LF Origin .1 .2 Zoom 5. 10

```
probid = 04/13/05 14:29:41
basis: XY
( 0.000000, 1.000000, 0.000000)
( 1.000000, 0.000000, 0.000000)
origin:
( 0.01, 5.00, 0.01)
extent = ( 8.00, 8.00)
```

\*

```
Edit cel 1
          Cell 1
xyz = 0.01, 5.00, 0.01
CURSOR SCALES 0 MWG MESH
PostScript ROTATE
COLOR wwn1:p
XY YZ ZX
LABEL off off
MBODY on
```

[Click here](#) or picture or menu



cel  
imp  
rho  
den  
vol  
fcl  
mas  
pwt  
mat  
tmp  
wnn  
ext  
pd  
dxc  
u  
lat  
fill  
ijk  
nonu  
pac  
tal  
PAR  
N

Redraw Plot> End

# Color Contour and Mesh Tally Plots

## HEU Cans in a Hex Lattice

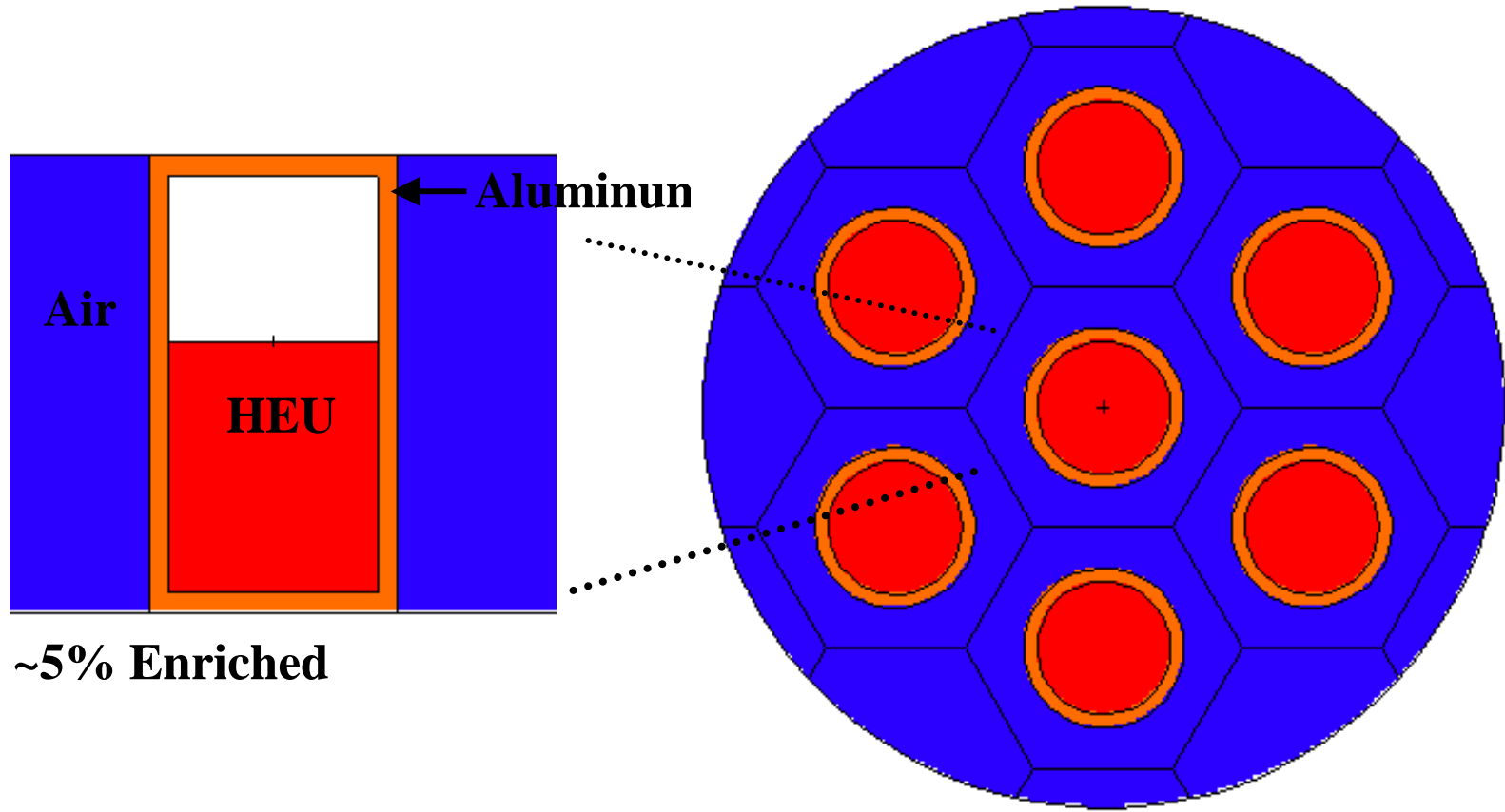
```
1 1 -8.4      -1      u=1      imp:n=1
2 0          -2      u=1      imp:n=1
3 2 -2.7      -3 1 2  u=1      imp:n=1
4 3 -.001     3      u=1      imp:n=1
10 3 -.001     -6 lat=2 u=2      imp:n=1 fill=-2:2 -2:2 0:0
      2 2 2 2 2 2 2 1 1 2 2 1 1 2 2 2 2 2 2 2
11 0          -8      imp:n=1 fill=2
50 0          8      imp:n=0

1 rcc 0 0 0 0 12 0 5
2 rcc 0 12 0 0 8 0 5
3 rcc 0 -1 0 0 22 0 6
6 rhp 0 -1 0 0 22 0 9 0 0
8 rcc 0 -1 0 0 22 0 30

m1 1001 5.7058e-2 8016 3.2929e-2 92238 2.0909e-3 92235 1.0889e-4
m2 13027 1
m3 7014 .8 8016 .2
rcode 10000 1 10 40
ksrc 0 6 0 18 6 0 -18 6 0 9 6 15 -9 6 15 9 6 -15 -9 6 -15
tmesh
  rmesh12
  cora12 -30. 53i 30.
  corb12 0. 12.
  corc12 -30. 35i 30.
endmd
```



# Color Contour and Mesh Tally Plots

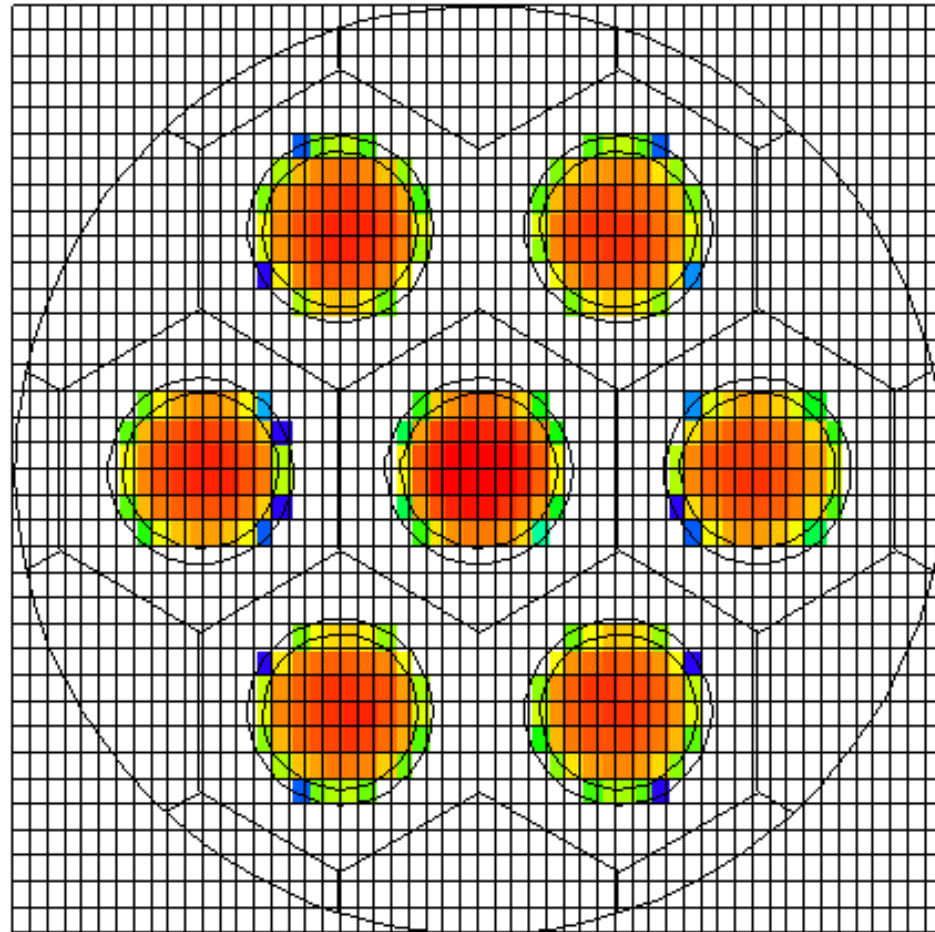


04/12/05 15:41:44  
cylinders containing critical  
fluid in macrobody hex lattice

UP RT DN LF Origin .1 .2 Zoom 5. 10

probid = 04/11/05 16:42:09  
basis: XZ  
( 1.000000, 0.000000, 0.000000)  
( 0.000000, 0.000000, 1.000000)  
origin:  
( 0.00, 5.00, 0.00)  
extent = ( 40.00, 40.00)

cel  
imp  
rho  
den  
vol  
fcl  
mas  
pwt  
mat  
tmp  
wnn  
ext  
pd  
dxc  
u  
lat  
fill  
ijk  
nonu  
pac  
tal  
PAR  
N



Edit cel 1  
Cell 1  
xyz = 0.00, 5.00, 0.00  
CURSOR SCALES 0 MT+Cell  
PostScript ROTATE  
COLOR tal12 LEVEL  
XY YZ ZX  
LABEL off off  
MBOY on

[Click here](#) or picture or menu

Redraw

Plot>

End

# User-Interface Enhancements – 2.5.0

- **Three other miscellaneous improvements**
  - READ card
  - HISTP card extension
  - DXTRAN/Detector underflow control

# User-Interface Enhancements

HEU Cans in a Hex Lattice

READ FILE=cells NOECHO

READ FILE=surfaces

```
m1      1001 5.7058e-2   8016 3.2929e-2
        92238 2.0909e-3  92235 1.0889e-4
m2      13027 1
m3      7014 .8 8016 .2
kcode  10000 1 10 40
ksrc   0 6 0   18 6 0   -18 6 0   9 6 15
        -9 6 15   9 6 -15   -9 6 -15
tmesh
  rmesh12
  cora12  -30. 53i 30.
  corb12   0. 12.
  corc12  -30. 35i 30.
endmd
```

File "cells"

```
1 1 -8.4      -1      u=1      imp:n=1
2 0           -2      u=1      imp:n=1
3 2 -2.7     -3 1 2   u=1      imp:n=1
4 3 -.001     3      u=1      imp:n=1
10 3 -.001    -6 lat=2 u=2    imp:n=1
      fill=-2:2 -2:2 0:0
      2 2 2 2 2
      2 2 1 1 2
      2 1 1 1 2
      2 1 1 2 2
      2 2 2 2 2
11 0          -8              imp:n=1 fill=2
50 0          8              imp:n=0
```

File "surfaces"

```
1 rcc 0 0 0 0 12 0 5
2 rcc 0 12 0 0 8 0 5
3 rcc 0 -1 0 0 22 0 6
6 rhp 0 -1 0 0 22 0 9 0 0
8 rcc 0 -1 0 0 22 0 30
```

# Physics Enhancements

- **Four model physics improvements**
  - Mix & match of libraries and models
  - CEM upgrade to 2K
  - INCL 4/ABLA physics models
  - Secondary-particle production

# Mix & Match of Libraries and Models

- **Mix and Match**

- 5<sup>th</sup> entry on PHYS:N (3<sup>rd</sup> on PHYS:H) is superseded.
  - Code will choose cutoff based on individual library's upper energy.
- MX card allows the user to substitute isotopes or specify model physics based on individual M card entries.
  - Isotope substitution can be done for nuclides where no table data is available.
  - No neutron data table exists for Ge. As-75 could be used where the material cards call for Ge (see example).
- Available for neutrons, protons, and photons (photonuclear) only.

# Mix & Match of Libraries and Models

Mix & Match for 100 MeV Neutrons => BGO Crystal

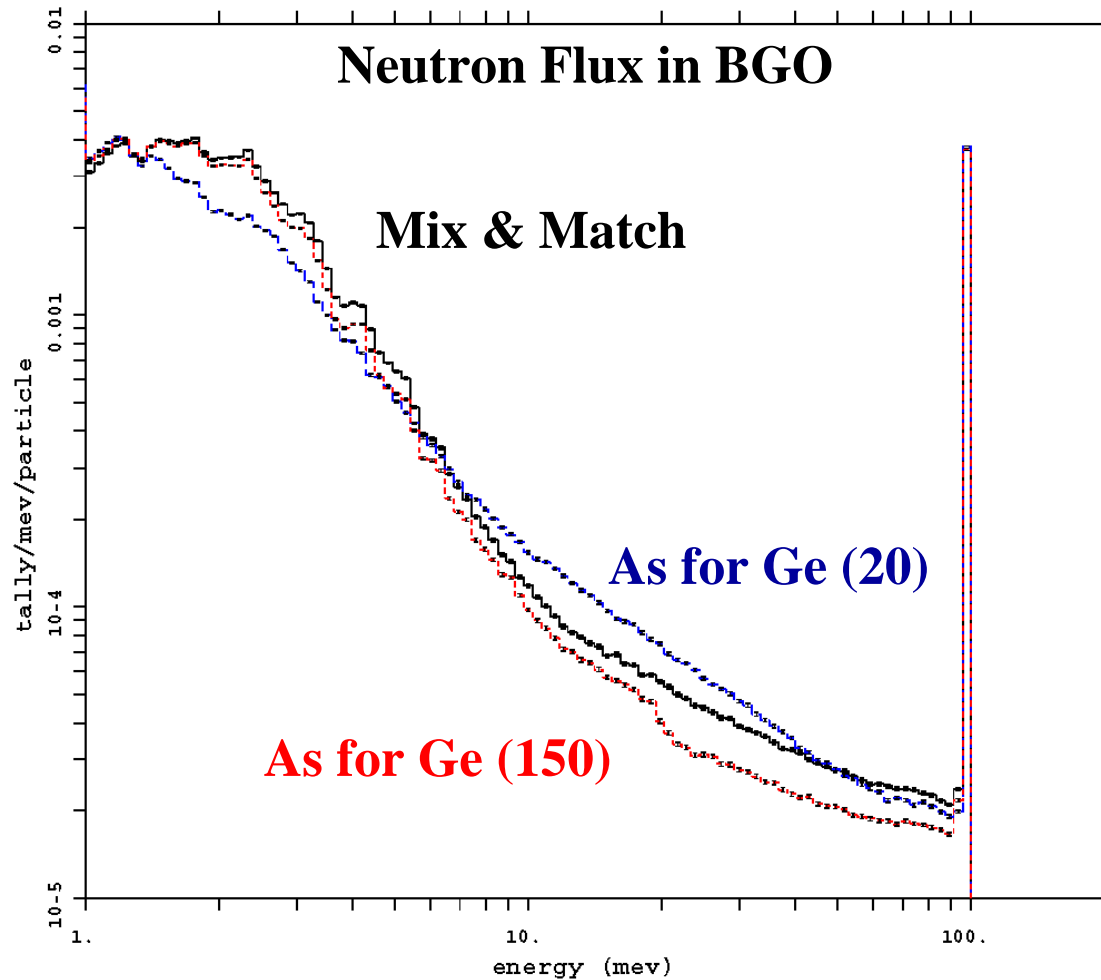
```
1 1 -7.130 -1 imp:n=1
2 0          1 imp:n=0

1 RCC 0 0 0 0 0 8.433 3.932

sdef sur=1.3 vec=0 0 1 dir=1 erg=100
nps 1000
m1      83209.24c 0.2105 8016.24c 0.6316 32000 0.1579
phys:n 101 3j -1
c      mx1:n      j              j              33075
c      phys:n 101 3j 20 $ Models above 20 MeV
c      mx1:n      j              j              33075
c      phys:n 101 3j 150 $ Models above 150 MeV
mode n p
f4:n 1
e0 1 100log 100
f21:p 1.2

m1      1002 1      1003 1      6012 1      20040 1
mx1:n   j          model      6000      20000
mx1:h   model     1001      j          j
mx1:p   6012      0          j          j
```

# Mix & Match of Libraries and Models





# CEM Upgrade to 2K (from 95)

- **CEM 2k**
  - Improvements in elementary cross sections, nuclear masses and pairing energies.
  - Algorithm improvements decrease time required for some problems by a factor of 6.
  - Utilizes measurements from the GSI experiments on isotope production.

# CEM Upgrade to 2K (from 95)

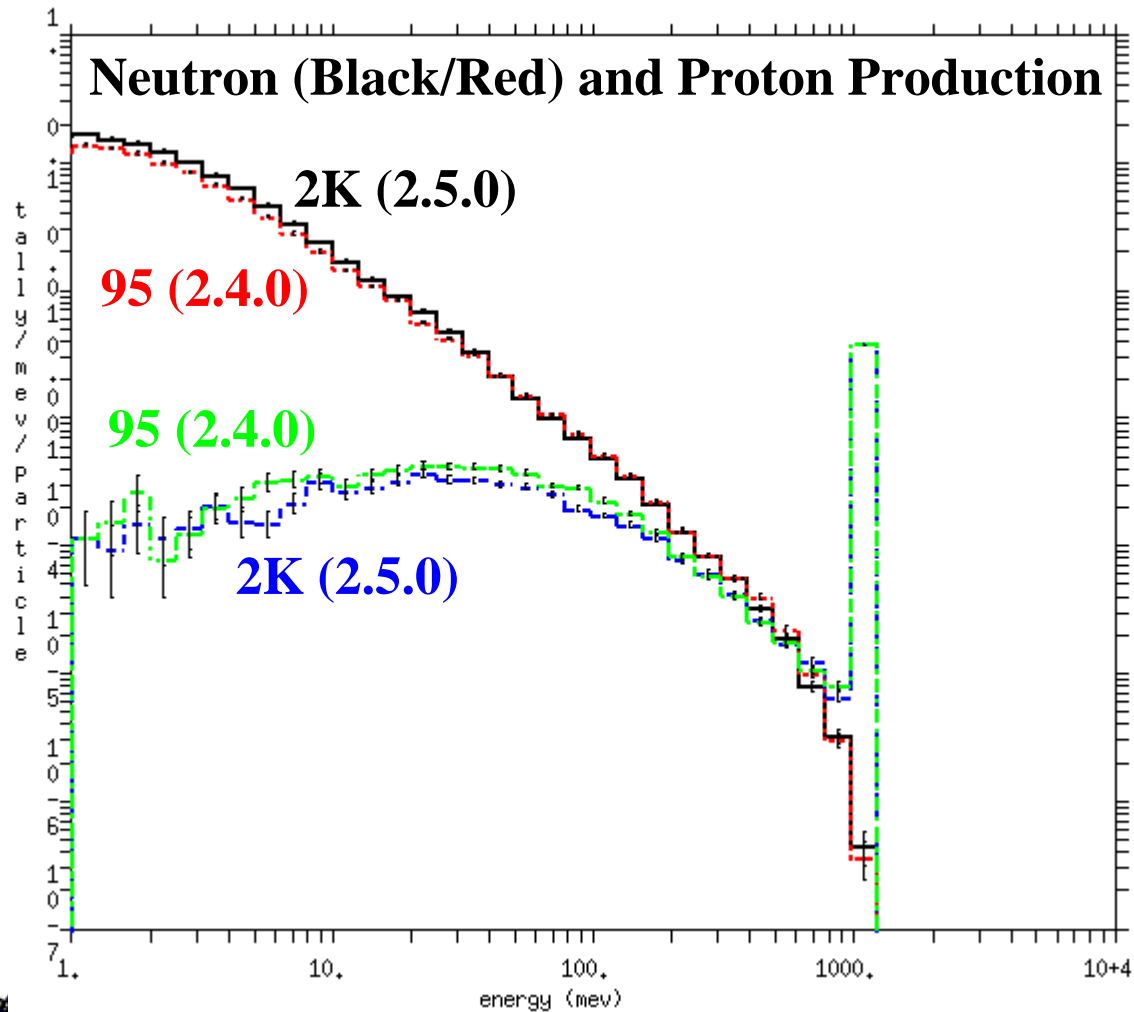
CEM for 1.2 GeV Protons => Pb

```
1 1 -11. -1 imp:h 1
2 0      1 imp:h 0

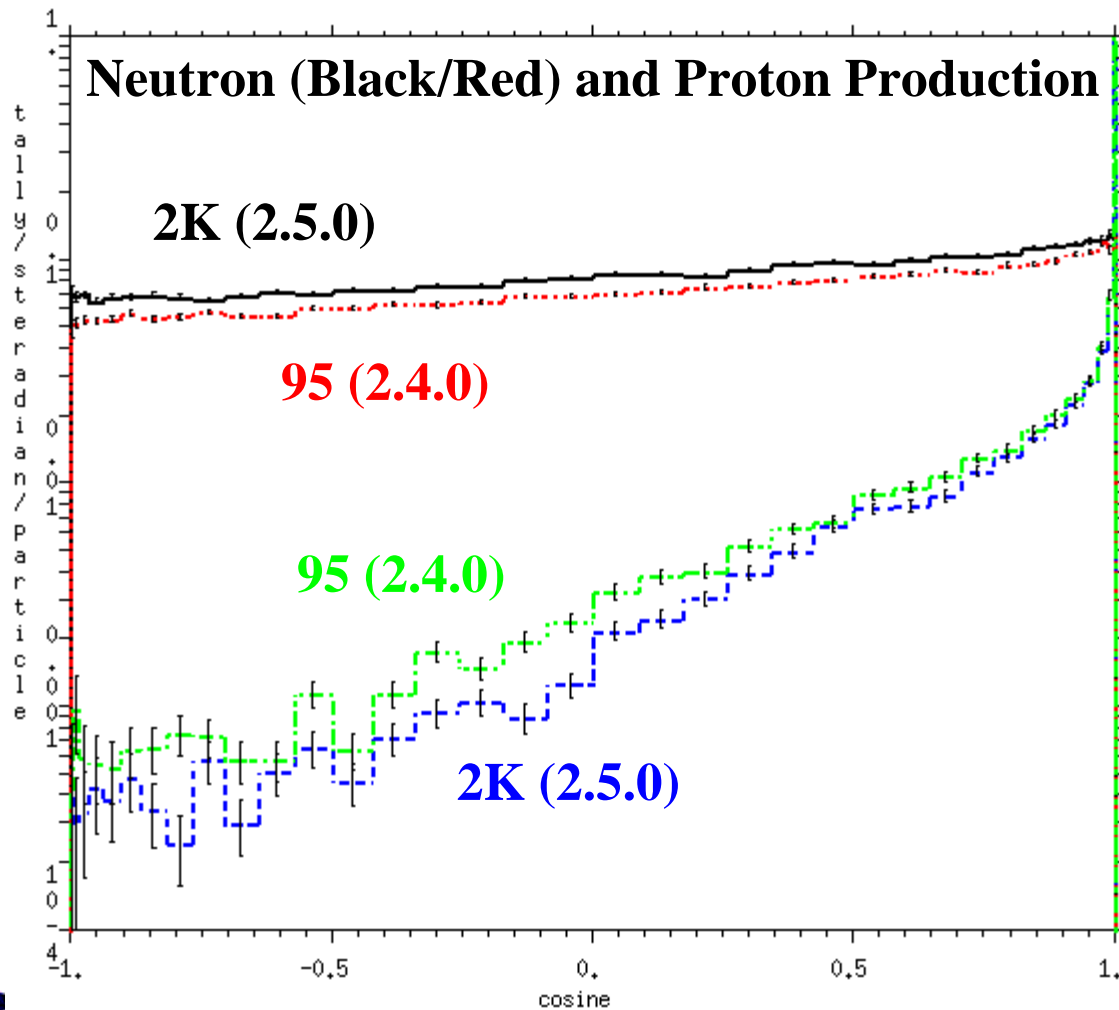
1 so 1.0

mode h n
sdef par h erg=1200 vec 1 0 0 dir 1
m1 82208 1
phys:h 1300 j 1
phys:n 1300 3j 1
nps 100000
lca      8j 1
f1:n     1
ft1      frv 1 0 0
*c1      175 34i 0
f11:n    1
e11      1. 30log 1200.
f21:h    1
ft21     frv 1 0 0
*c21     175 34i 0
f31:h    1
e31      1. 30log 1200.
```

# CEM Upgrade (version 95 to 2K)



# CEM Upgrade (version 95 to 2K)



# INCL 4/ABLA Physics Models

- **INCL 4/ABLA is a combined intranuclear cascade/evaporation model.**
  - User can specify combinations of existing models and INCL 4/ABLA (see example).
  - Implemented by agreement with CEA
  - Primarily useful in the 200 MeV – 2 GeV range.
  - Somewhat slower than Bertini and CEM2K.

# INCL 4/ABLA Physics Models

Various Physics Models for 1.2 GeV Protons => Pb

```
1 1 -11. -1 imp:h 1
2 0      1 imp:h 0
```

```
1 so .01
```

```
mode h n
```

```
sdef par h erg=1200 vec 0 0 1 dir 1
```

```
m1 82208 1
```

```
phys:h 1300 j 0
```

```
phys:n 1300 3j 0
```

```
nps 100000
```

```
f1:n 1
```

```
ft1 frv 0 0 1
```

```
*c1 167.5 9i 17.5 0 T
```

```
e1 1 50log 1300
```

```
LCA 7j -2 0 $ Bertini/Dresner
```

```
LCA 7j -2 0 $ Bertini/ABLA
```

```
LEA 6J 2
```

```
LCA 2j 2 4j -2 $ ISABEL/Dresner
```

```
LCB 4j 1300
```

```
LCA 2j 2 4j -2 $ ISABEL/ABLA
```

```
LCB 4j 1300
```

```
LEA 6J 2
```

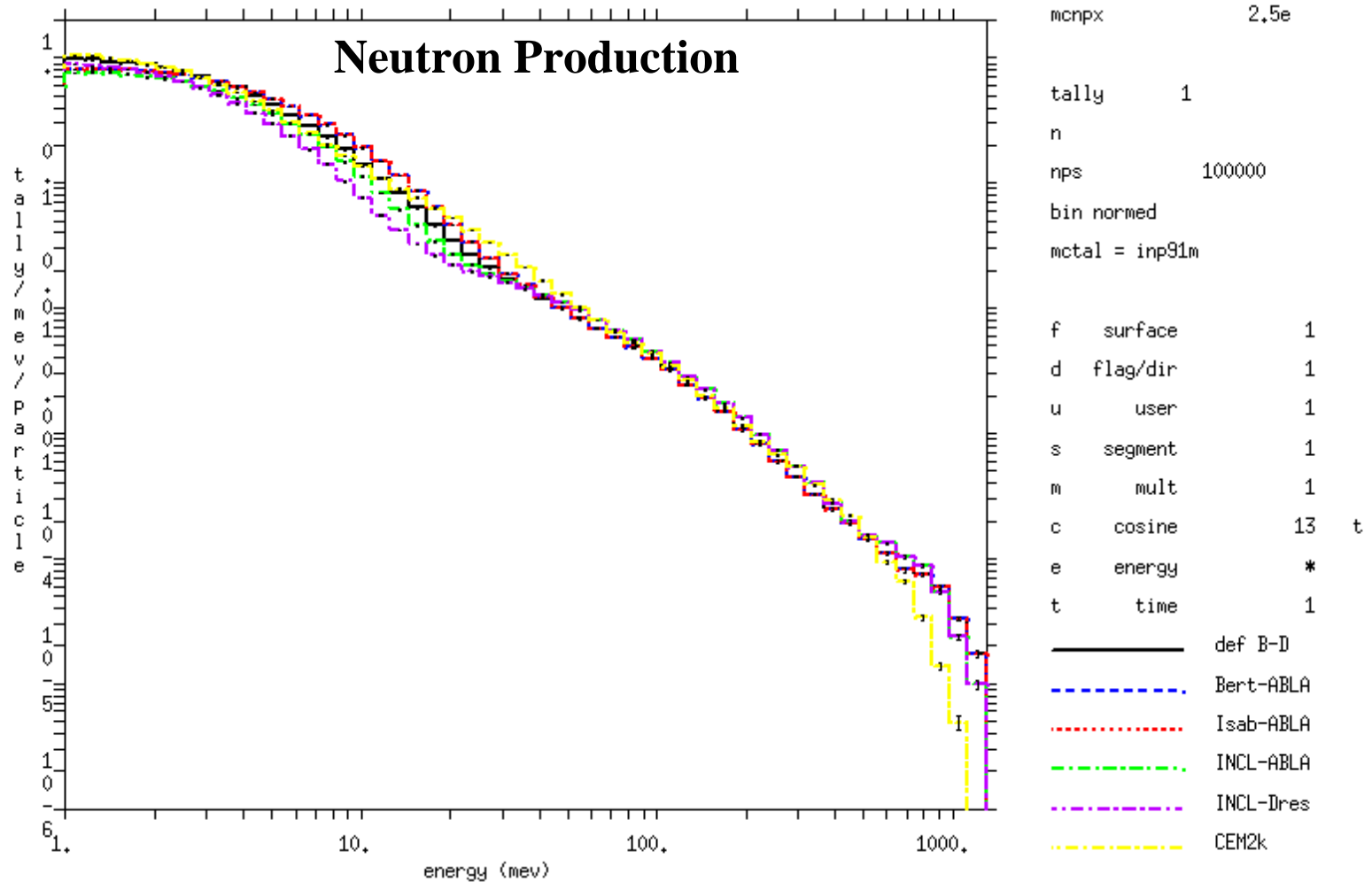
```
LCA 7j -2 2 $ INCL4/ABLA
```

```
LEA 6j 2
```

```
LCA 7j -2 2 $ INCL4/Dresner
```

```
LCA 7j -2 1 $ CEM2K
```

# Secondary-Particle Production



# Physics Enhancements - 2.5.0

- **Two neutron physics improvements**
  - Fission multiplicity
  - $S(\alpha, \beta)$  secondary-energy smoothing



# Fission Multiplicity

- **Implements more precise distribution of neutrons per fission event.**
  - Previous method used linear sampling between the two integers that bracket  $\nu$ .
  - New method implements a sampling based on specific nuclei data or a Gaussian sampling on user-specified value.
  - Controlled by 6<sup>th</sup> entry on PHYS:N card.
  - Specifically will improve sampling for coincidence neutron populations.

# Fission Multiplicity

## U-235 Fission Multiplicity in H2O

```
1 1 -1.0 -1 imp:n=1
2 0 1 imp:n=0
```

```
1 SPH 0 0 0 0.1
```

```
sdef
```

```
m1 1001 2 8016 1 92235 1.e-1
```

```
c phys:n 5j 0
```

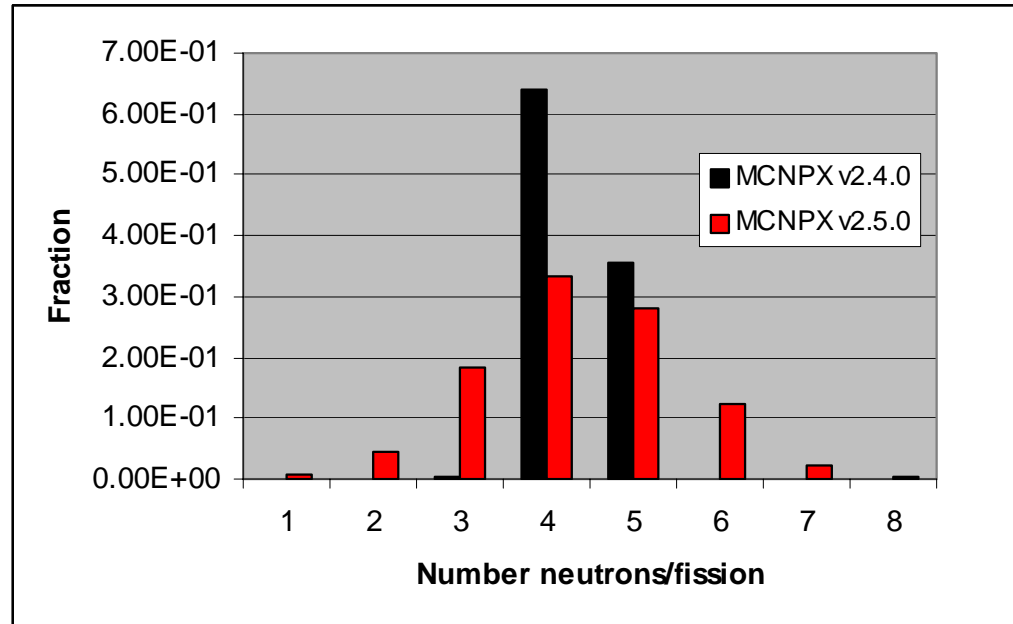
```
phys:n 5j -1
```

```
print
```

```
nps 5000000
```

# Fission Multiplicity

14 MeV Neutrons on U-235



# Physics Enhancements – 2.5.0

- **Three photon physics improvements**
  - Photonuclear physics model
  - Photon Doppler broadening
  - Variance reduction with pulse-height tallies

# Photonuclear Physics Model

Photonuclear 10 MeV Photons => Pb

```
1 1 -7.86 -1 imp:n=1
2 0 1 imp:n=0
```

```
1 SPH 0 0 0 2
```

```
mode n p
sdef par=p erg=10.0
```

```
phys:p 3j 1
```

```
m1 82208 1
```

```
c mx1:p model
```

```
nps 1000000
```

```
f1:n 1
```

```
e1 1e-3 50log 10.
```

Photonuclear 10 MeV Photons => U-235

```
1 1 -7.86 -1 imp:n=1
2 0 1 imp:n=0
```

```
1 SPH 0 0 0 2
```

```
mode n p
sdef par=p erg=10.0
```

```
phys:p 3j 1
```

```
m1 92235 1
```

```
xs1 92235.27u 233.024994 bofod01u 0 1 54868 2946 0 0 0.0
```

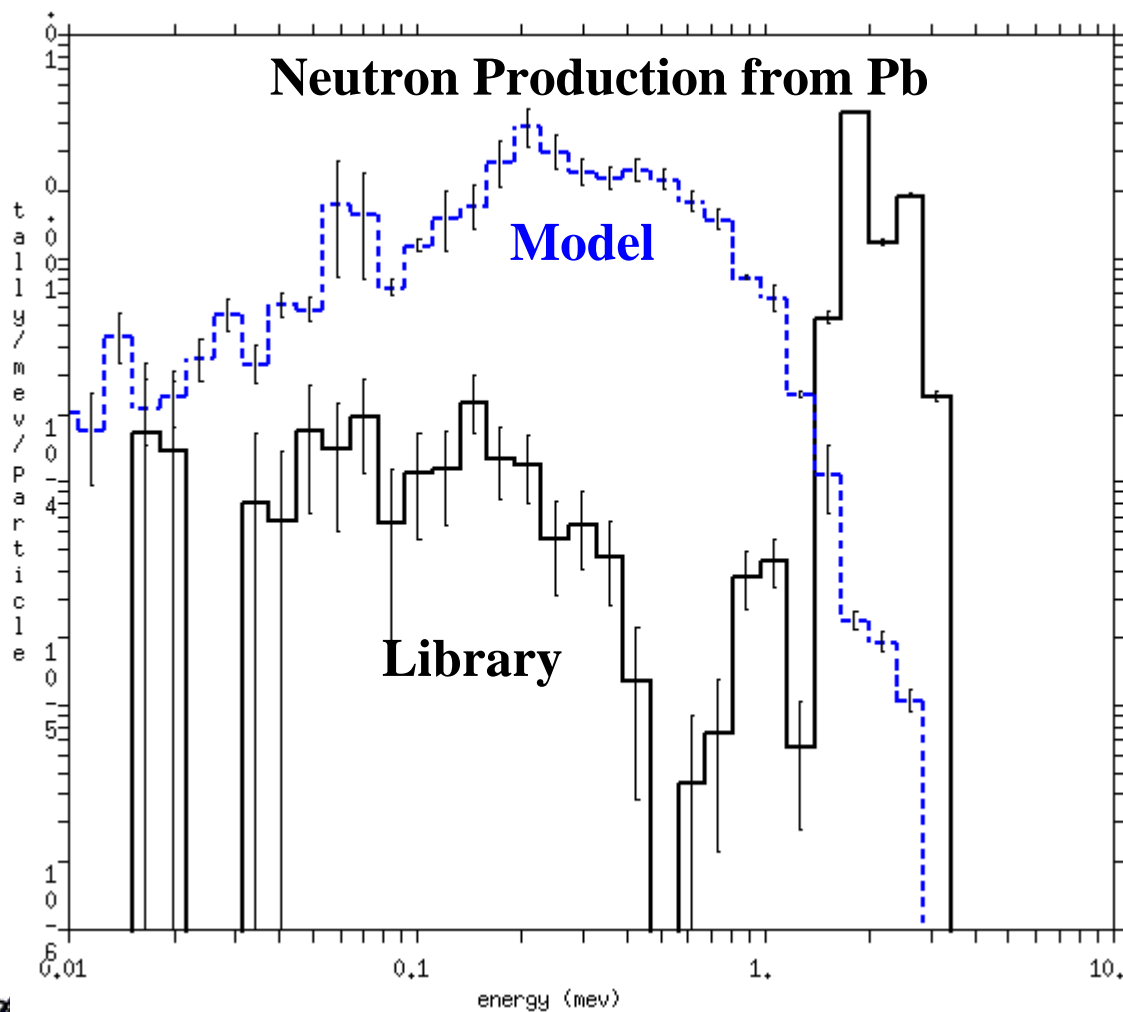
```
c mx1:p model
```

```
nps 1000000
```

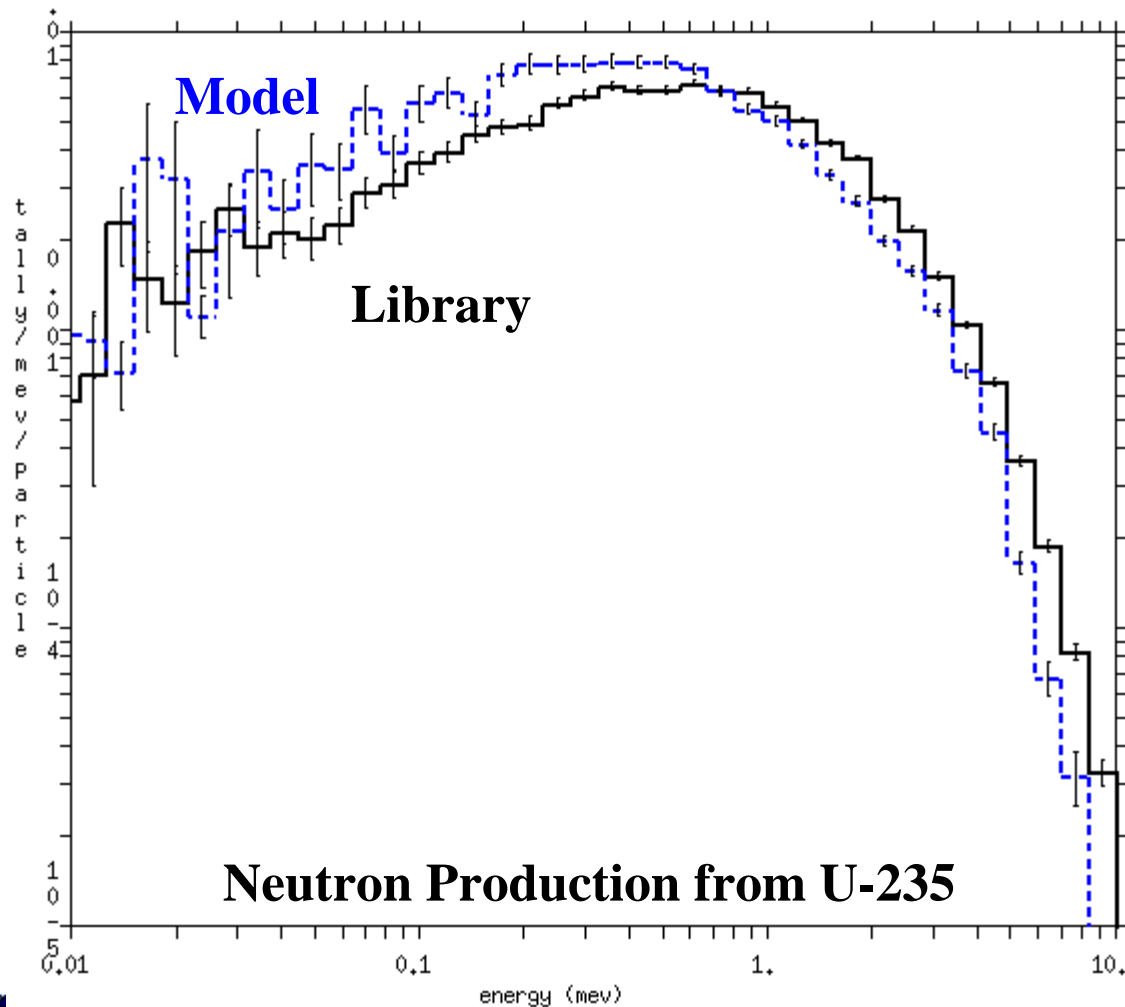
```
f1:n 1
```

```
e1 1e-3 50log 10.
```

# Photonuclear Physics Model



# Photonuclear Physics Model

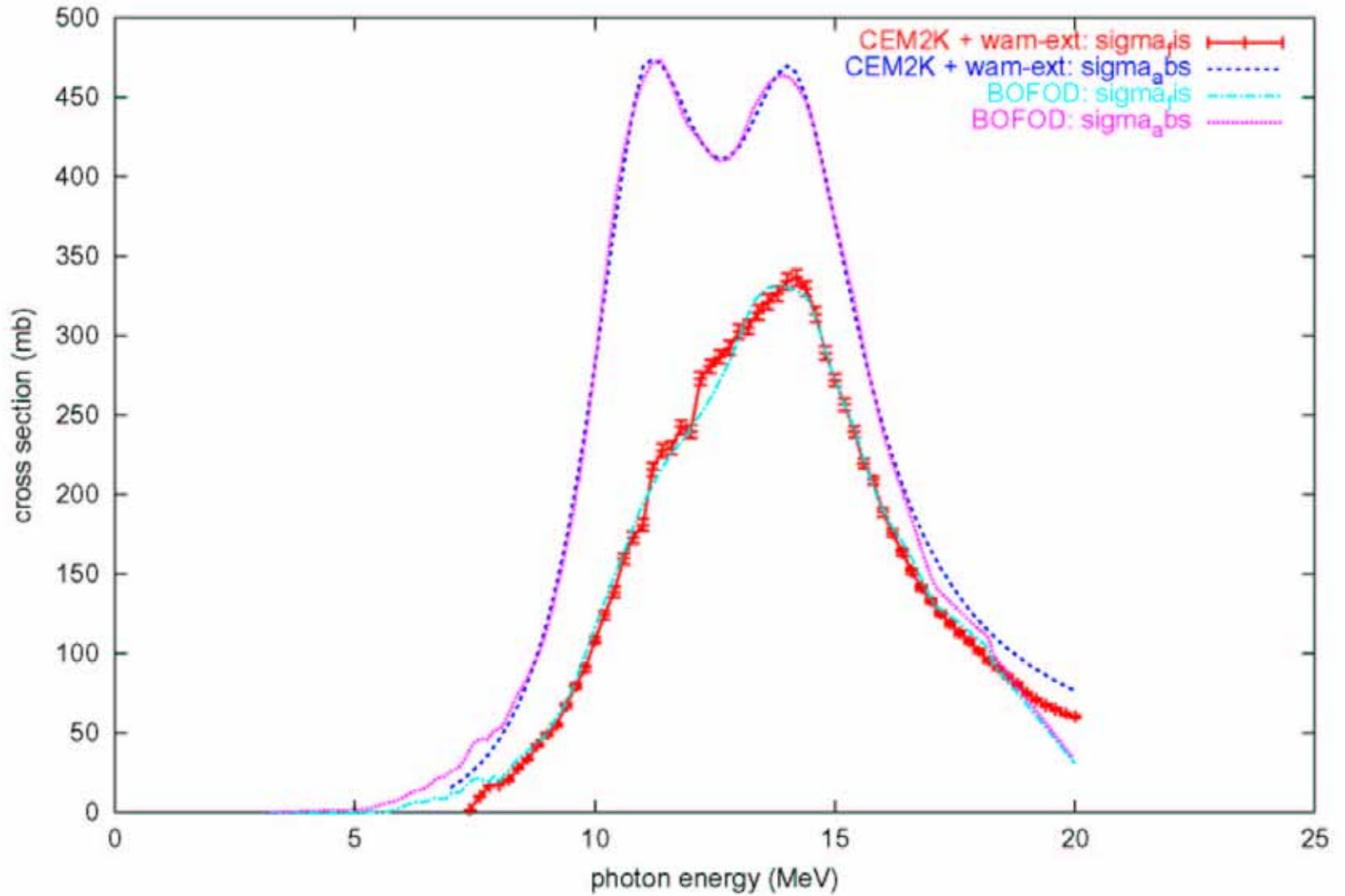


# Photonuclear Capabilities

- Libraries available for some nuclides
  - H, C, O, Al, Si, Ca, Fe, Cu, Ta, W, Pb
  - Feature implemented in 2000
- Models available for all nuclides
  - Provided with the CEM2K INC package (April 2003)
  - Actinide GDR parameters recently improved
- User can control use of libraries vs models
  - Default is to use libraries, otherwise models
  - Biasing available to enhance secondary production



U-235 Fission Cross Section in Photon Induced Reactions



# Photon Doppler Broadening

DB for 100 keV Photons => C

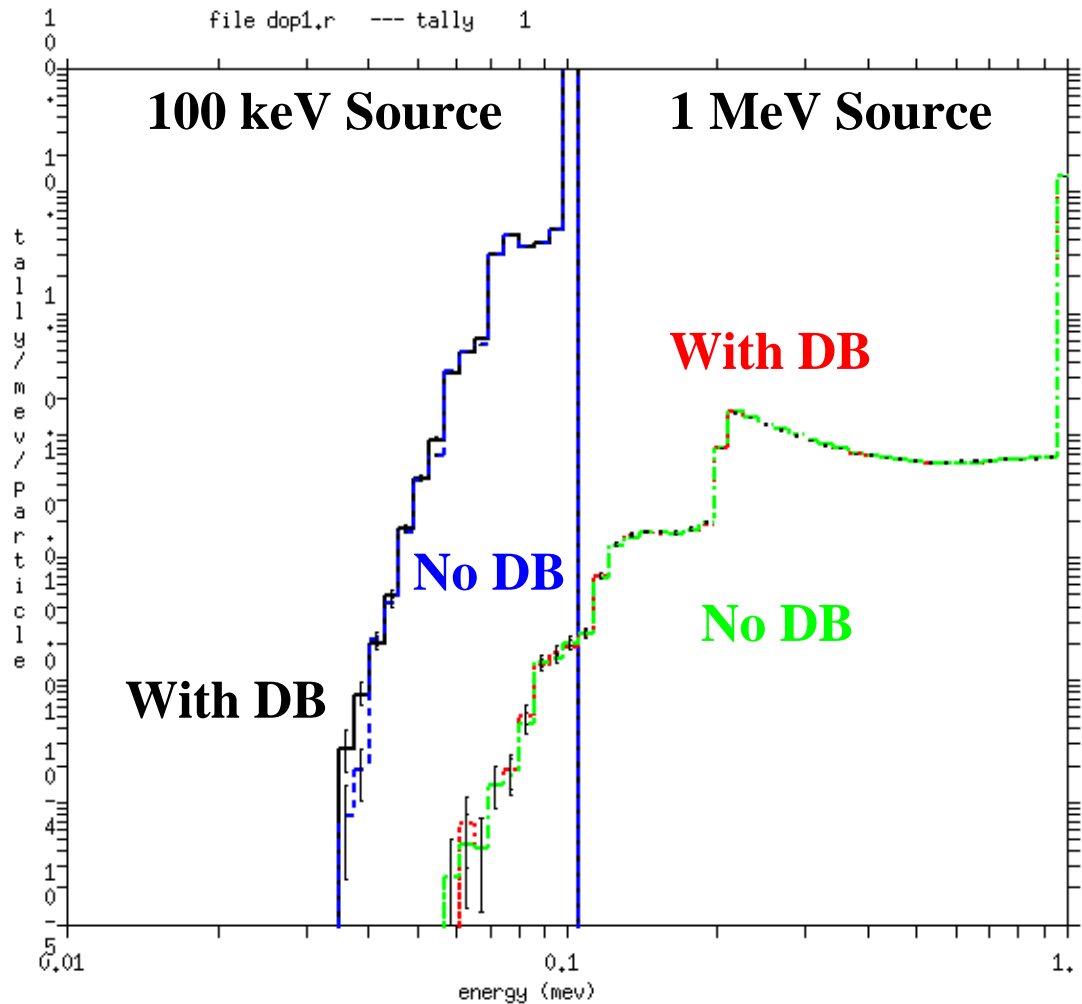
```
1 1 -1.00 -1 imp:p=1
2 0 1 imp:p=0
```

```
1 SPH 0 0 0 1
```

```
mode p
sdef par=p erg=0.1
vec=1 0 0 dir=1
```

phys:p j 1 1 j 0

```
m1 6000 1
nps 10000000
fl:p 1
e1 1e-2 99log 1
```



# Variance Reduction with Pulse-Height Tallies

- **Pulse-Height tally Variance Reduction has been a goal in Monte-Carlo codes for many years.**
  - Implemented in MCNPX v2.5.0 and can result in dramatic speed improvements.
  - Deconvolves the particle “trees” to get correct PHT.
  - Not all variance reduction techniques supported.
  - Unsupported VRTs result in a fatal error.

# Variance Reduction with Pulse-Height Tallies

## VRT with PHT 1 MeV Photons Incident

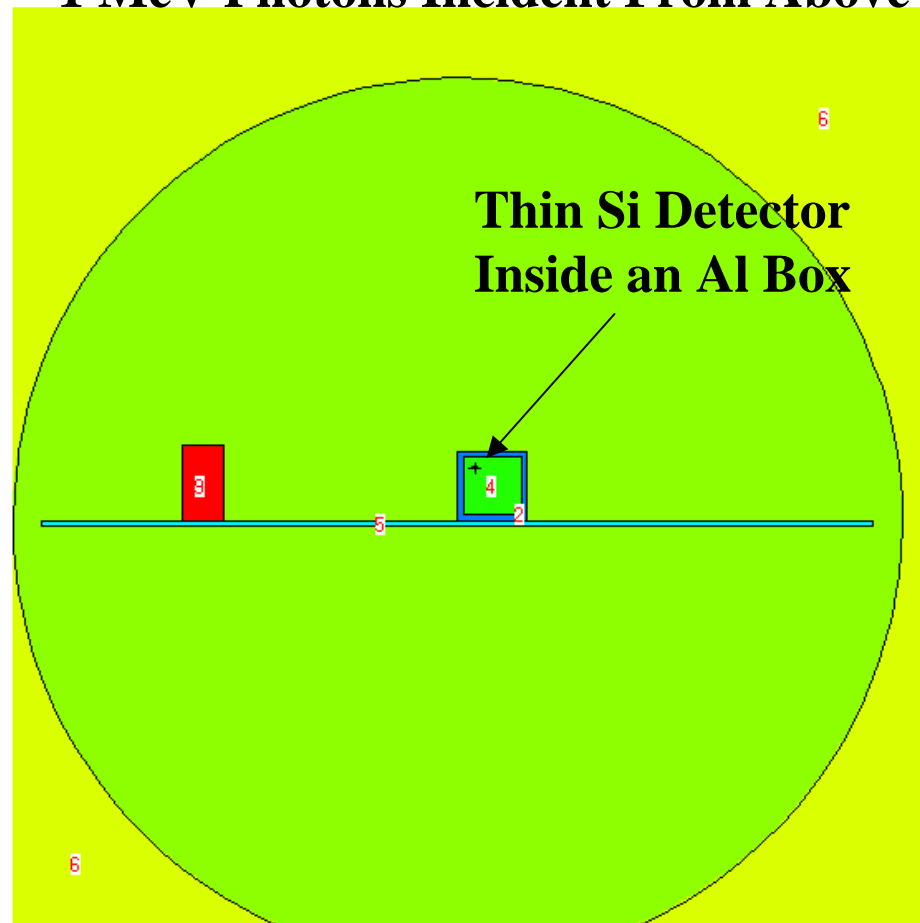
```
1 2 -2.33 -1      imp:p,e 1
2 1 -2.7  -2 5    imp:p,e 1
3 1 -2.7  -3      imp:p,e 1
4 1 -1.35 -5 20   imp:p,e 1
5 0 2 3 -4 6 7 8 imp:p,e 1
6 0 4              imp:p,e 0
7 1 -2.7 -6       imp:p,e 1
8 1 -2.7 -7       imp:p,e 1
9 1 -2.7 -8       imp:p,e 1
20 1 -1.35 -20 21 imp:p,e 1
21 1 -1.35 -21 1  imp:p,e 1

1 rcc 3 3 9 0 0 0.25 0.4
2 box 0 0 0 12 0 0 0 12 0 0 0 12
5 box 1 1 1 10.0 0 0 0 10.0 0 0 0 10.0
3 rcc 0 0 -1 0 0 1.0 72.0
4 sph 0 0 0 77.0
6 rcc -9 44 0 0 0 16 5
7 rcc 30 30 0 0 0 12 10
8 rcc 9 -44 0 0 0 13 7
20 sph 3 3 9.1 1.8
21 sph 3 3 9.1 .9
```

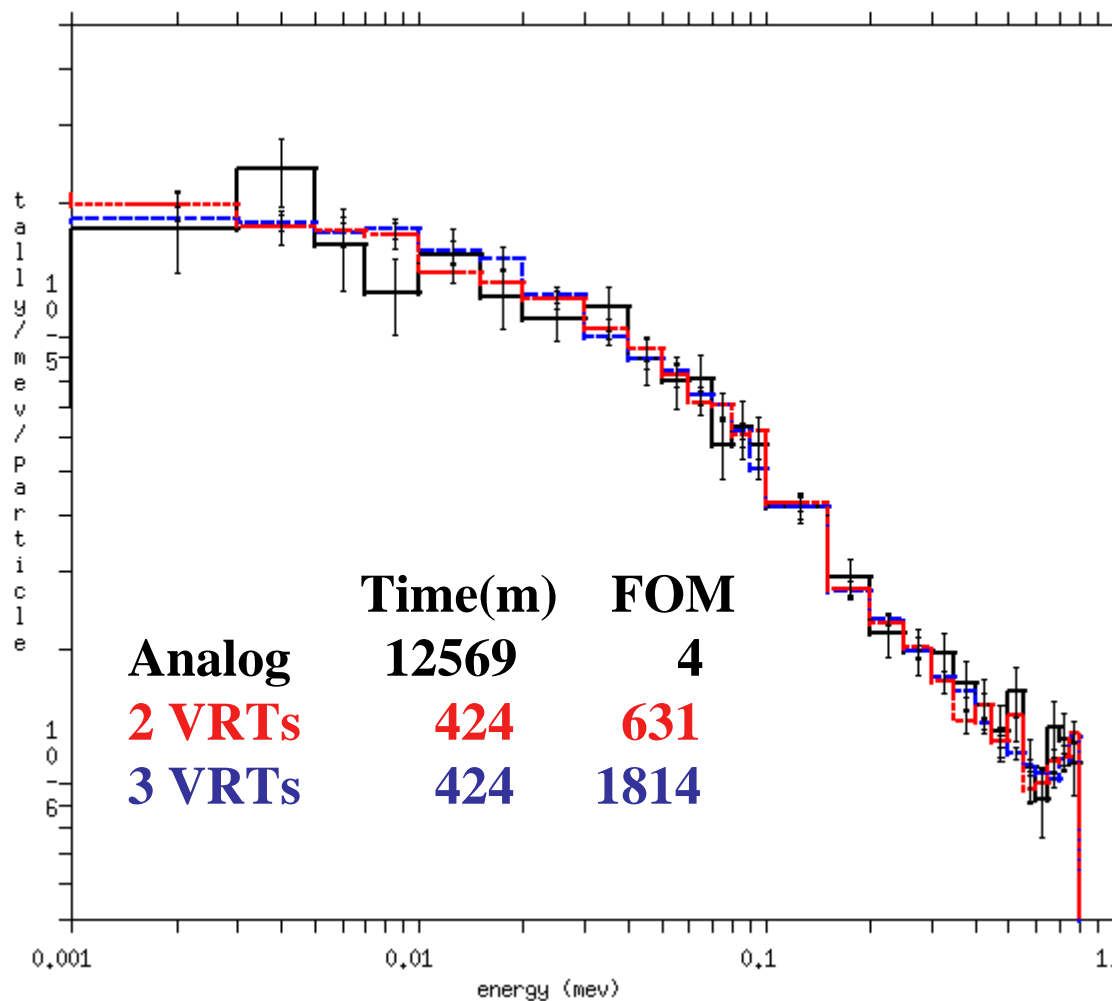
```
wwp:p 100 4 4
wnw1:p .0002 .002 .02 .001 .02 -1
      .02 .02 .02 .0005 .0002
fcl:p -1 0 7r -1 -1
mode p e
m1 13027 1 $ (2.7 g/cc) Al
m2 14028 1 $ Si (detector element, 2.33 g/cc)
sdef erg=1 pos=0 0 21 par=p rad=d1 ext=0
      axs=0 0 1 vec=0 0 -1 dir=1
si1 0 72.0
sp1 -21 1
phys:p
phys:e 7j 0
f8:p 1
e8 0 1.e-5 .001 2i .007 .01 .015 .02
   7i .1 19i 1.1
nps 100000000
```

# Variance Reduction with Pulse-Height Tallies

1 MeV Photons Incident From Above



# Variance Reduction with Pulse-Height Tallies

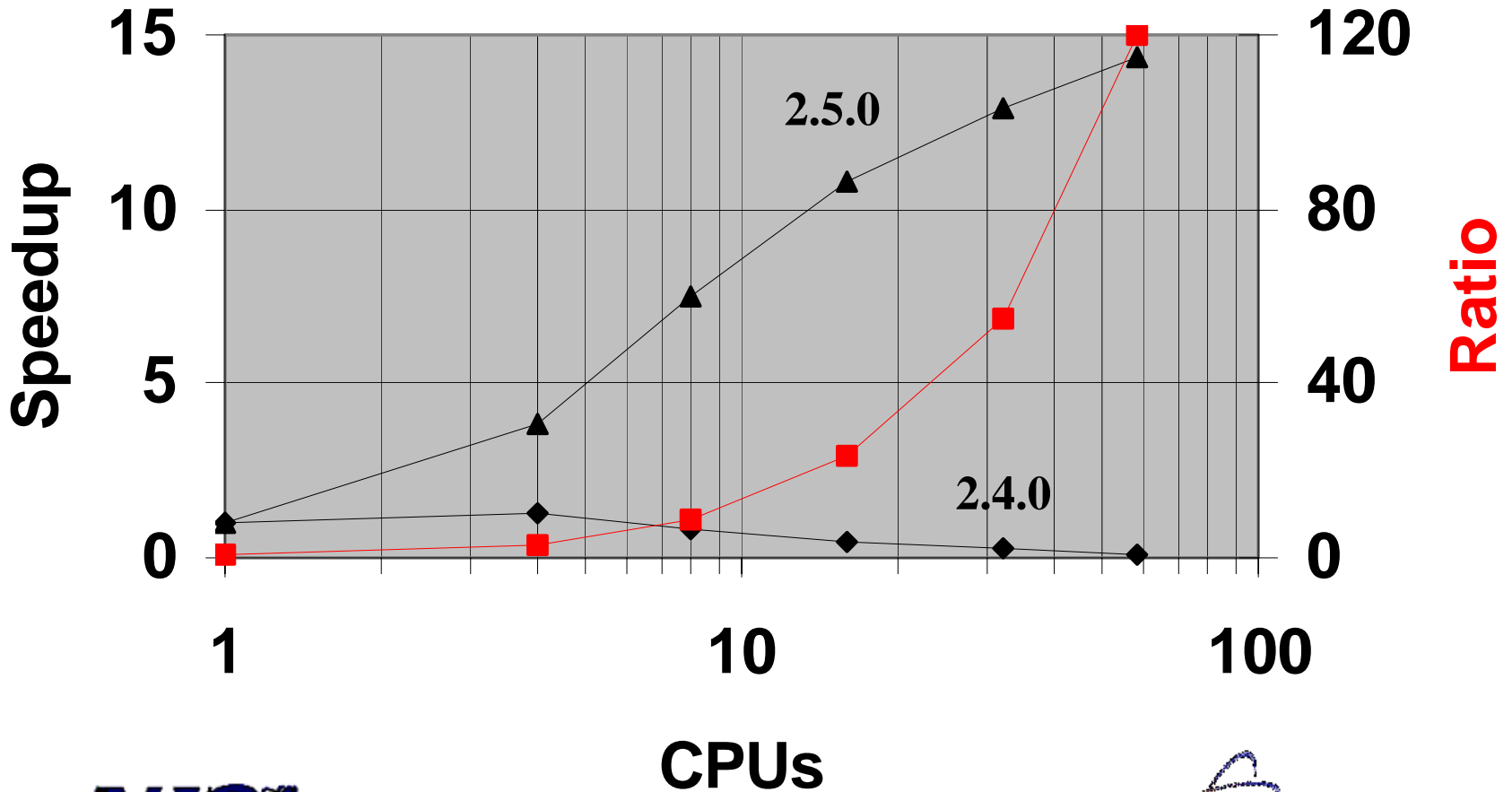


# Infrastructure Enhancements – 2.5.0

- **8-byte integers**
  - Users can now run billions of particles
  - Often required for parallel calculations
  - Runs about 20% slower on most systems
- **Support for new compilers**
  - Mac OS X with IBM compiler
  - Windows PC and Linux with Intel compiler
- **Parallel processing with MPI**
  - PVM option is still available
- **MPI speedup for criticality problems**
  - Eliminates collection of fission source after each cycle

# Godiva Run on a Linux Cluster

500K Particles/Cycle for 200 Cycles





# New Features - MCNPX Version 2.6.A

- **BURN card - transmutation using Cinder90**
  - **M. Fensin's paper in session Best of RPSD**
- **Long file names (40 vs. 8 characters)**
- **STOP card - terminate tallies at desired precision**
- **Corrections/enhancements/extensions**
  - **Proton step size control (HSTEP on M card)**
  - **New  $S(\alpha,\beta)$  scattering law**
  - **Differential data tallies extended to table physics**
  - **Separate printout of induced fission multiplicity**

# BURN Card

Burn 7 fuel pins surrounded by H2O in a hex lattice

```

1 1 -8.3 -1 u=1 imp:n=1 vol=192.287 $ Fuel
2 8 -6.5 1 -2 u=1 imp:n=1 $ Clad
3 9 -0.7 2 u=1 imp:n=1 $ Water
100 9 -1.8 -3 u=8 lat=2 imp:n=1 fill=-2:2 -2:2 0

```

```

8 8 8 8 8
8 8 1 1 8
8 1 1 1 8
8 1 1 8 8
8 8 8 8 8

```

```

101 0 -4 imp:n=1 fill=8
102 9 -1.8 4 -5 imp:n=1
103 0 5 imp:n=0

```

```

1 rcc 0 0 0 0 0 365 0.4095
2 rcc 0 0 -1 0 0 367 0.4750
3 rhp 0 0 -1 0 0 367 0.6565 0 0
4 rhp 0 0 -1 0 0 367 0 1.895 0
*5 rhp 0 0 -1.1 0 0 367.2 0 1.896 0

```

```

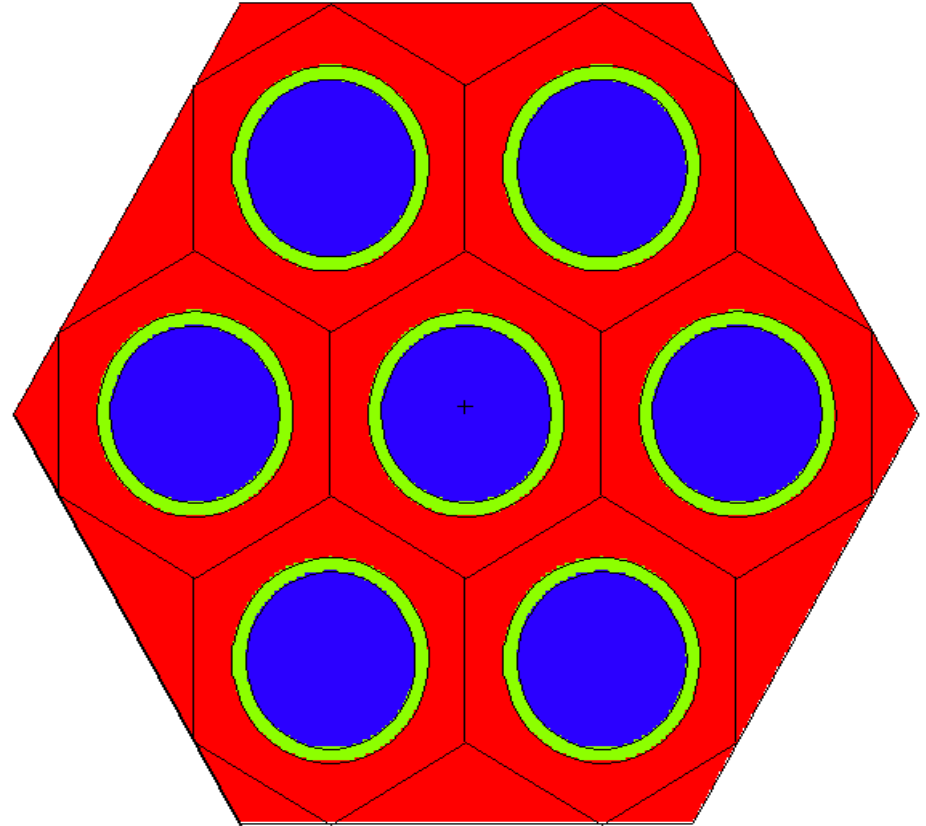
BURN TIME=50,500
MAT=1
POWER=0.066956
PFRAC=1.0,1.0
OMIT=1,6,6014,7016,8018,9018,90234,91232
BOPT=1.0, 4

```

```

m1 8016.60c 2.0 92235.60c 0.01 92238.60c 0.9
m8 40000.60c 1.0
m9 1001.60c 2.0 8016.60c 1.0
mt9 lwtr.01t
kcode 100 1.0 10 50
ksrc 0 0 150 1.313 0 150 -1.313 0 150 0.6565 1.137 150
0.6565 -1.137 150 -0.6565 1.137 150 -0.6565 -1.137 150

```



nuclides with atom fractions below 1.000E-10 for a material are zeroed and deleted from print tables after t=0

neutronics and burnup data

step	duration (days)	time (days)	power (MW)	keff	flux	ave. nu	ave. q	burnup (Gwd/MTU)
0	0.000E+00	0.000E+00	6.696E-02	0.99763	3.641E+14	2.449	200.981	0.000E+00
1	5.000E+01	5.000E+01	6.696E-02	1.00012	3.701E+14	2.554	203.154	2.383E+00
2	5.000E+02	5.500E+02	6.696E-02	0.85037	4.638E+14	2.869	209.385	2.621E+01

actinide inventory for sum of materials at end of step 2, time 5.500E+02 (days), power 6.696E-02 (MW)

no.	zaid	mass (gm)	activity (Ci)	sp. act. (Ci/gm)	atom den. (a/b-cm)	atom fr.	mass fr.
1	92234	3.465E-04	2.154E-06	6.217E-03	4.636E-09	2.577E-07	2.533E-07
2	92235	3.935E-01	8.506E-07	2.161E-06	5.244E-06	2.914E-04	2.877E-04
3	92236	1.789E+00	1.157E-04	6.467E-05	2.374E-05	1.319E-03	1.308E-03
4	92237	7.849E-03	6.405E+02	8.160E+04	1.037E-07	5.763E-06	5.739E-06
5	92238	1.355E+03	4.553E-04	3.361E-07	1.782E-02	9.905E-01	9.904E-01
6	92239	1.539E-03	5.158E+04	3.351E+07	2.016E-08	1.121E-06	1.125E-06
14	94242	1.117E+00	4.418E-03	3.954E-03	1.446E-05	8.034E-04	8.169E-04
	totals	1.368E+03	1.041E+05	7.610E+01	1.799E-02	1.000E+00	1.000E+00

nonactinide inventory for sum of materials at end of step 2, time 5.500E+02 (days), power 6.696E-02 (MW)

no.	zaid	mass (gm)	activity (Ci)	sp. act. (Ci/gm)	atom den. (a/b-cm)	atom fr.	mass fr.
1	6012	2.336E-06	0.000E+00	0.000E+00	6.096E-10	1.638E-08	1.186E-08
2	6013	1.057E-02	0.000E+00	0.000E+00	2.545E-06	6.839E-05	5.366E-05
3	8016	1.891E+02	0.000E+00	0.000E+00	3.702E-02	9.946E-01	9.599E-01
4	8017	1.405E-02	0.000E+00	0.000E+00	2.588E-06	6.954E-05	7.132E-05
16	60145	3.469E-01	1.426E-14	4.112E-14	7.497E-06	2.014E-04	1.761E-03
	totals	1.970E+02	8.830E+01	4.483E-01	3.722E-02	1.000E+00	1.000E+00

# Long File Names

```
E:\MCNPX\scratch>.\mcnpx inp=test_long_names.txt na=test_long_names.  
mcnpx ver=26bc1 ld=Sat Jul 01 08:00:00 MST 2006 11/12/06 20:08:04
```

...

```
dynamic storage =      0 words,      0 bytes.  cp0 = 0.00  
run terminated when    10 particle histories were done.  
dump  2 on file test_long_names.r nps =      10 coll =      0  
      ctm = 0.00  nrn =      40  
mcrun is done
```

```
E:\MCNPX\scratch>dir
```

Directory of E:\MCNPX\scratch

```
11/12/2006 08:08 PM <DIR>      .  
11/12/2006 08:08 PM <DIR>      ..  
09/11/2006 03:23 PM      6,574,080 mcnpx.exe  
11/12/2006 08:08 PM      22,513 test_long_names.d  
11/12/2006 08:08 PM      25,510 test_long_names.o  
11/12/2006 08:08 PM      401,342 test_long_names.r  
08/04/2006 01:21 PM      481 test_long_names.txt  
5 File(s)  7,023,926 bytes  
2 Dir(s)  31,759,495,168 bytes free
```

# STOP Card

14 MeV neutrons in water - test STOP card

c cell cards

1 1 -1. -1 IMP:N=1  
2 0 1 IMP:N=0

1 so 30.0

m1 1001.60c 2. 8016.60c 1.

mt1 hh2o.20t

xs1 hh2o.20t 0.998623 ct00 0 1 1 1237501 0 0 2.530E-08

sdef erg=14.1

e0 1.00000E-11 625log 1.44544E+01

vol 1.

f44:n 1

f141:n 1

STOP F44 .01 NPS 10000 CTME 10.0

OUTPUT FILE

ltally fluctuation charts

	tally 44					tally 141				
nps	mean	error	vov	slope	fom	mean	error	vov	slope	fom
1000	5.6661E+01	0.0144	0.0031	10.0	489705	3.7459E-01	0.0354	0.0003	0.0	81225
2000	5.6824E+01	0.0103	0.0016	10.0	468574	3.7000E-01	0.0254	0.0002	10.0	77294
3000	5.7028E+01	0.0083	0.0010	10.0	477186	3.6737E-01	0.0209	0.0001	10.0	75590

\*\*\*\*\*  
dump no. 2 on file stop.r nps = 3000 coll = 599417 ctm = 0.03

# $S(\alpha, \beta)$ Secondary-Energy Smoothing

- **Improvement in  $S(\alpha, \beta)$  algorithm in code**
  - User interface is unchanged but underlying secondary energy treatment is improved.
  - The structure in the  $S(\alpha, \beta)$  data is often in discrete energies resulting in non-physical spikes in the thermal neutron spectrum.
  - New algorithm smooths over non-physical structure in data to provide smoother and better low-energy neutron spectra.

# S( $\alpha, \beta$ ) Secondary-Energy Smoothing

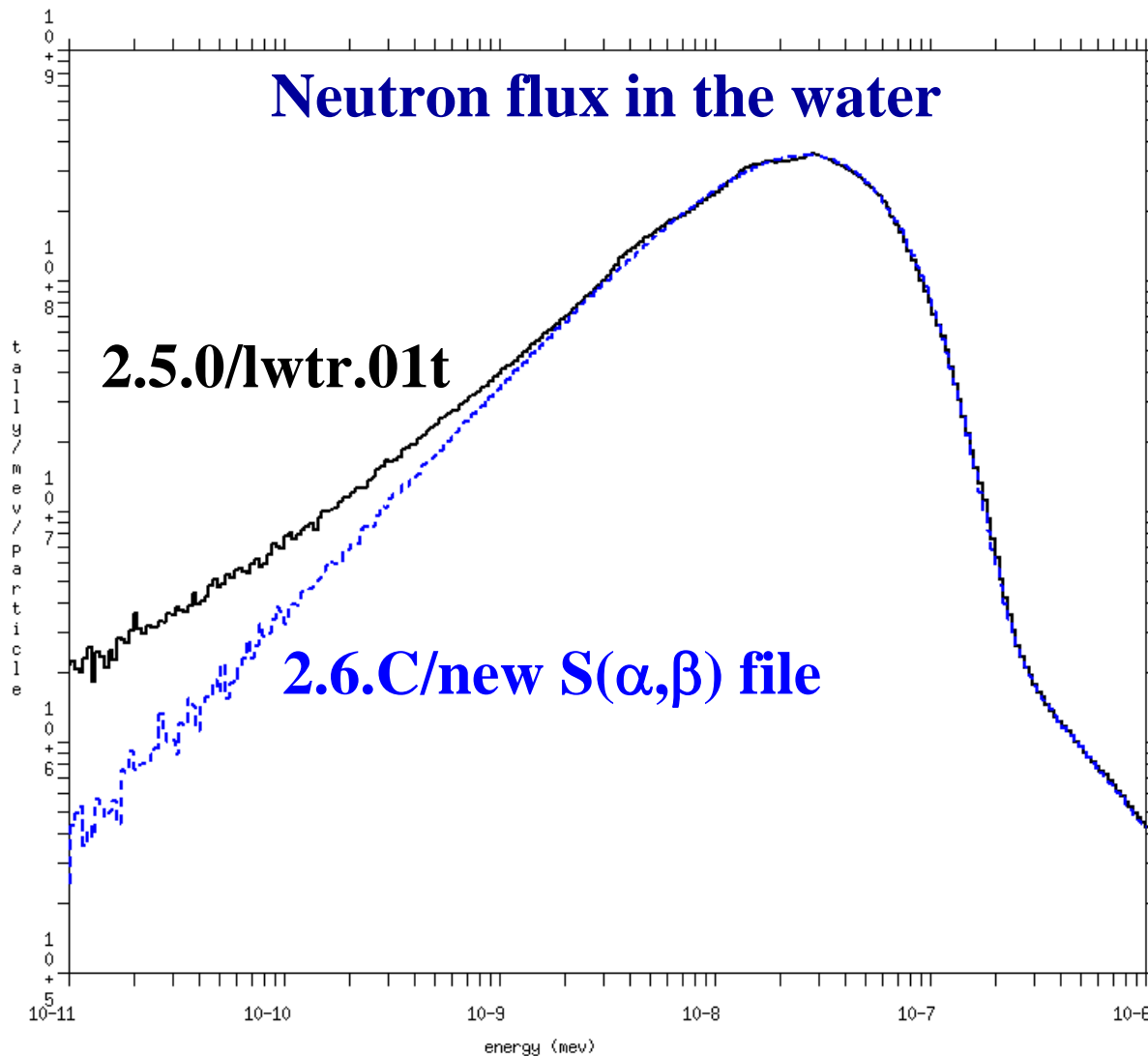
## S(alpha,beta) Smoothing for HEU Cans in a Hex Lattice

```
1 1 -8.4      -1      u=1      imp:n=1
2 0           -2      u=1      imp:n=1
3 2 -2.7      -3 1 2  u=1      imp:n=1
4 3 -.001     3      u=1      imp:n=1
10 3 -.001    -6 lat=2 u=2      imp:n=1 fill=-2:2 -2:2 0:0
      2 2 2 2 2 2 2 1 1 2 2 1 1 1 2 2 2 2 2 2 2 2 2 2 2 2
11 0          -8      imp:n=1 fill=2
50 0          8      imp:n=0

1 rcc 0 0 0 0 12 0 5
2 rcc 0 12 0 0 8 0 5
3 rcc 0 -1 0 0 22 0 6
6 rhp 0 -1 0 0 22 0 9 0 0
8 rcc 0 -1 0 0 22 0 30

m1 1001 5.7058e-2 8016 3.2929e-2
    92238 2.0909e-3 92235 1.0889e-4
mt1 lwtr.01t
m2 13027 1
m3 7014 .8 8016 .2
kcode 10000 1 10 40
ksrc 0 6 0 18 6 0 -18 6 0 9 6 15 -9 6 15 9 6 -15 -9 6 -15
f4:n (1<(10[-2:2 -2:2 0:0])<11)
sd4 1
e4 1e-11 50log 1e-6
```

# New $S(\alpha,\beta)$ Treatment





# New Features – Version 2.6.B

- **Transmutation improvements**
  - **Predictor/corrector**
  - **Automatic selection of FP dist. (thermal, fast, high)**
- **CEM INC model upgrade (from 2K to 03)**
- **FIELD card–planetary gravity effects for neutrons**
- **Corrections/enhancements/extensions**
  - **New photon emission data: PHTLIB**
  - **Geometry plot basis vectors**
  - **Extend ZAID identifiers**

# CEM Upgrade

1 GeV protons into Fe-56

```
1 1 -7.86 -1 IMP:N=1
2 0 1 IMP:N=0
```

```
1 so 1.0
```

```
mode n h
m1 26056.24c 1.
nps 200000
sdef erg=1000 par=h
PHYS:N 1001.0
```

LCA 7j -2 1 \$ Use CEM

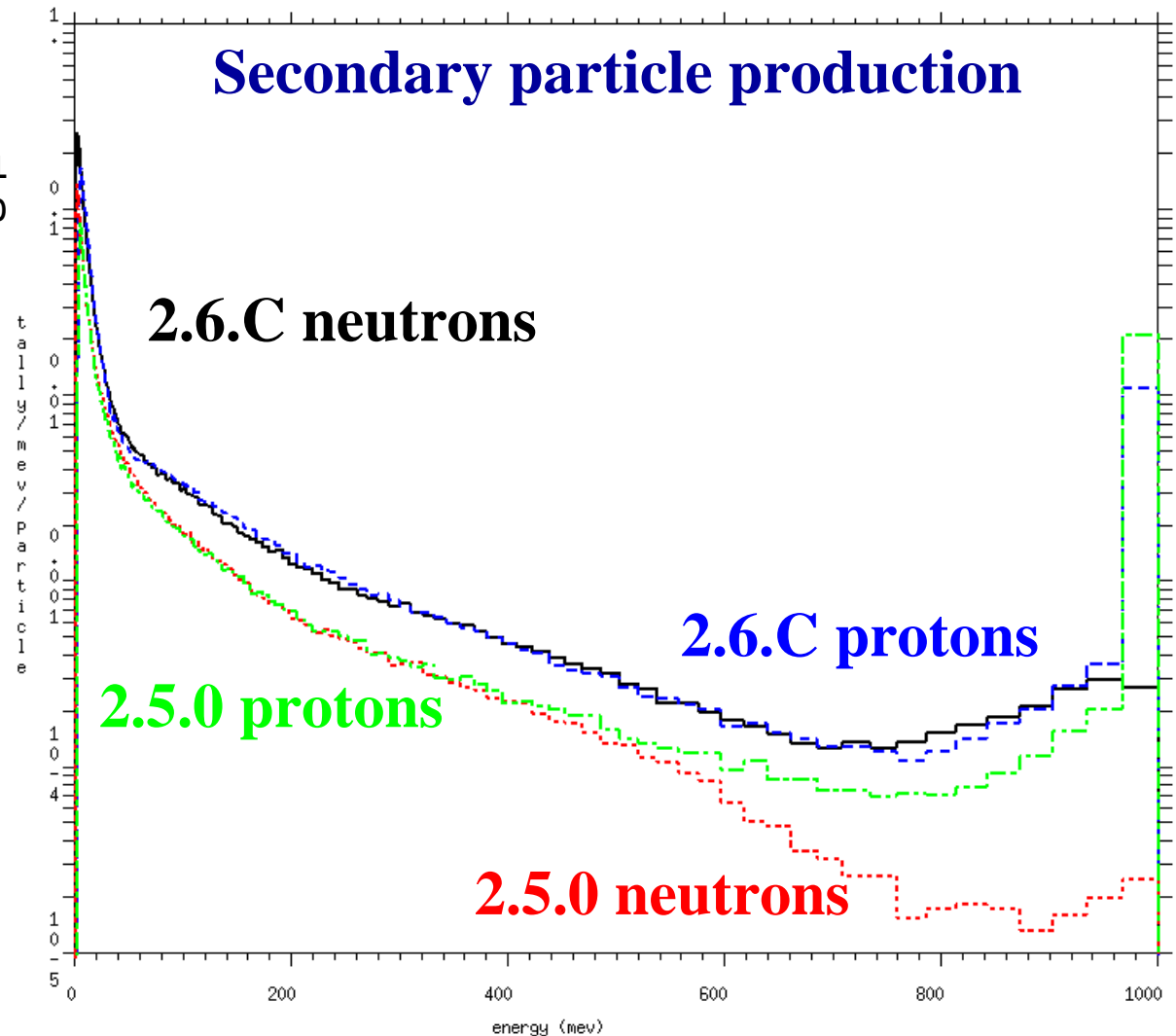
```
e0 1 199log 1000
```

```
c0 0 1
```

```
f1:n 1
```

```
f11:h 1
```

```
prdmp 2j 1
```



# CEM Upgrade

## Light-product yields ( $A < 30$ )

Model	Proton energy (MeV)					
	300	500	750	1000	1500	2600
BERTINI	1035	26.1	50.5	13.8	4.93	3.35
ISABEL	---	256	49.1	17.0	5.99	4.02
INCL	233	215	51.5	38.1	26.1	12.1
CEM2K	---	12.6	21.1	7.83	4.87	4.02
<b>CEM03</b>	<b>13.0</b>	<b>2.23</b>	<b>1.32</b>	<b>1.49</b>	<b>1.58</b>	<b>1.72</b>

## Heavy-product yields ( $A > 30$ )

Model	Proton energy (MeV)						Ave. Dev.
	300	500	750	1000	1500	2600	
BERTIN I	2.24	2.29	2.75	2.86	3.16	3.20	4.37
ISABEL	3.75	2.85	3.02	2.63	2.85	3.01	4.24
INCL	4.72	3.24	3.14	3.13	3.35	3.54	7.14
CEM2K	2.74	2.54	2.62	2.76	2.92	3.20	3.55
<b>CEM03</b>	<b>1.84</b>	<b>1.89</b>	<b>1.89</b>	<b>1.92</b>	<b>2.04</b>	<b>3.17</b>	<b>2.26</b>

**Mean-squared deviation factors between model predictions and experimental data measured at ITEP.**

# FIELD Card

5 GeV protons into Martian soil with gravity reflection

```
1      1 -1.0      -1      imp:n=1
100    2 -1.35e-5  -101 +1    imp:n=1
101    2 -1.28e-5  -102 +101  imp:n=1
102    2 -1.22e-5  -103 +102  imp:n=1
103    2 -1.14e-5  -104 +103  imp:n=1
104    2 -1.08e-5  -105 +104  imp:n=1
105    2 -1.01e-5  -106 +105  imp:n=1
999    0           +106      imp:n=0
```

```
1      so 339000000.0
101    so 339060000.0
102    so 339110000.0
103    so 339180000.0
104    so 339240000.0
105    so 339310000.0
106    so 339380000.0
```

```
m1     8016.60c -0.6 14000.60c -0.3 26056.60c -0.1
m2     6000.60c -0.27 7014.60c -0.02 8016.60c -0.70 18000.35c -0.01
```

FIELD GCUT=0.1320 GPAR=1 GRAD=3393.0 GSUR=106

mode h n p z / d t s a

lca 8j 1 \$ Use CEM

sdef par=9 erg=5000 sur=106 nrm=-1

nps 10000

print

phys:n 5010 j j j 20

e11 0. 1024i 10. 5000.

```
full1 0. 8016.00051 8016.00052 8016.00053 8016.00102 8016.
      14028.14027 14028.14026 14028.13027 14028.13026 14000.
      26056.00051 26056.00052 26056.00053 26056.00102 26056.
```

f11:p 1

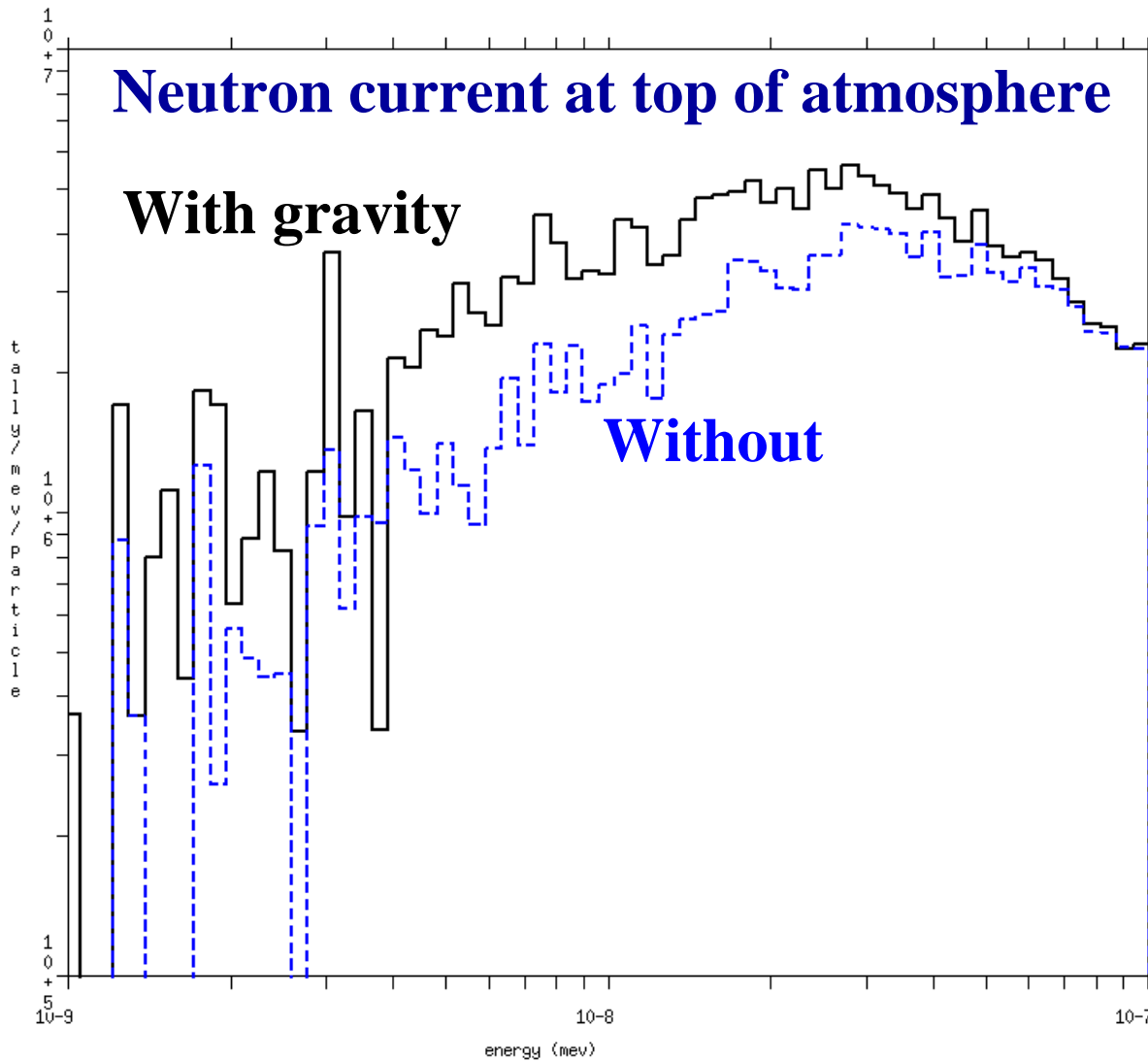
ft11 tag 1

e21 1e-10 99log 1e-7

f21:n 105

Undocumented Feature

# FIELD Card



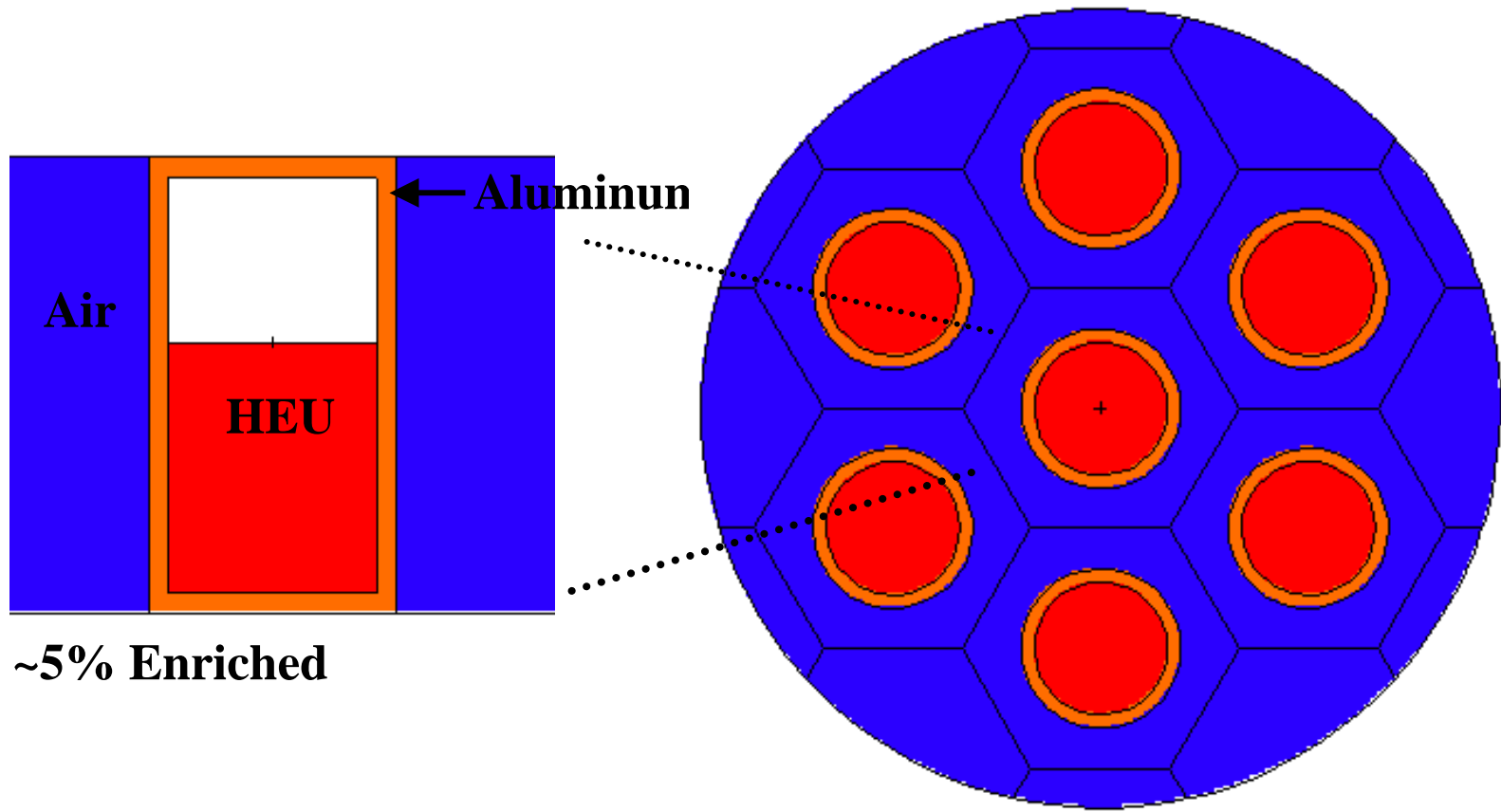
# New Features – Version 2.6.C

- **Eigenfunction convergence improvement**
- **Transmutation improvements**
  - Support for continue-runs
  - Printing of reaction rates sent to Cinder90
  - Reduced memory requirements
- **Spherical weight windows**
- **Delayed neutrons & gammas**
  - ~1000 nuclides treated with gamma line data
- **Photon tally tagging**
- **Model treatment for library absorption reactions**

# Eigenfunction Convergence

- ADS / LANL ADS reactor application
- Before - eigenfunction exhibits false convergence within fissile regions
  - Fission source produced by power iteration method
  - Can have a significant effect on burnup and shielding results
  - Can only be overcome by running more particles/cycle
- Now - fission source distribution is biased to minimize false convergence
  - Fission matrix is tabulated and split into symmetric/asymmetric parts
  - Asymmetric component is dampened to minimize statistical variations
  - Biasing parameters are derived and used in the next cycle
  - **Increases eigenfunction convergence by factors of 10-100**

# 7-Can HEU Test Problem

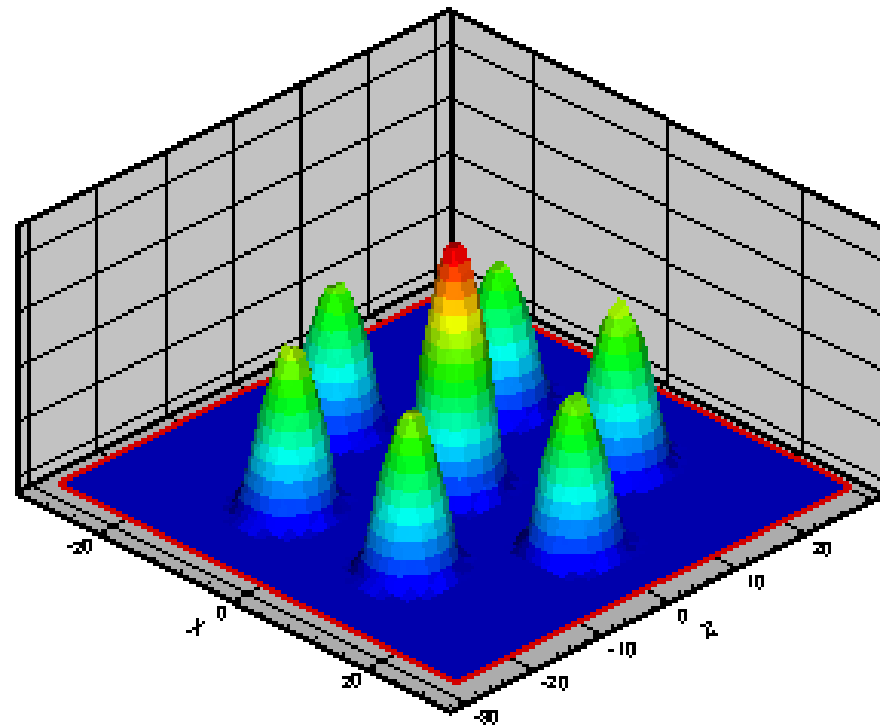
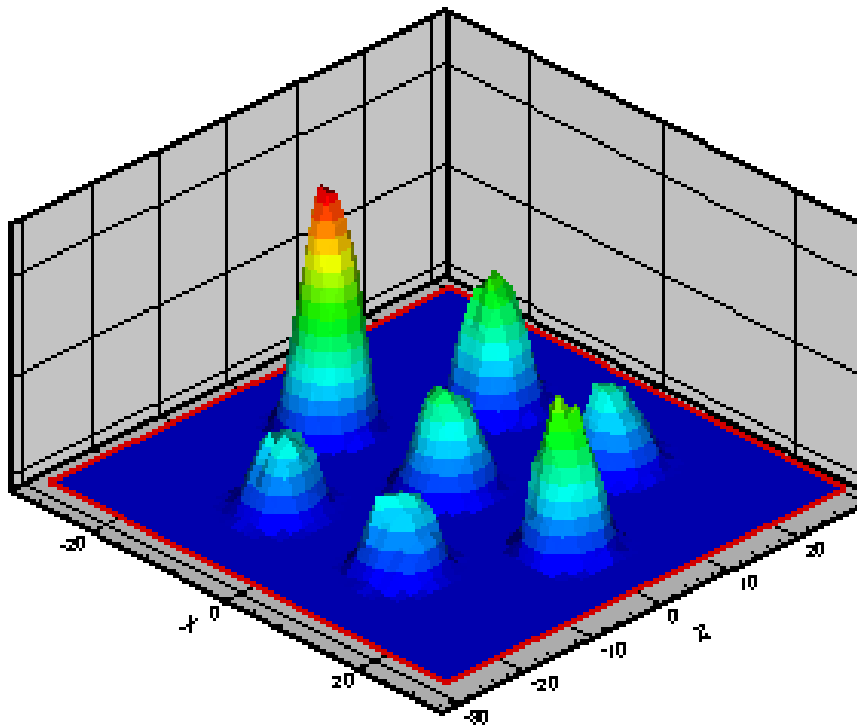




# Eigenfunction from the Standard Method

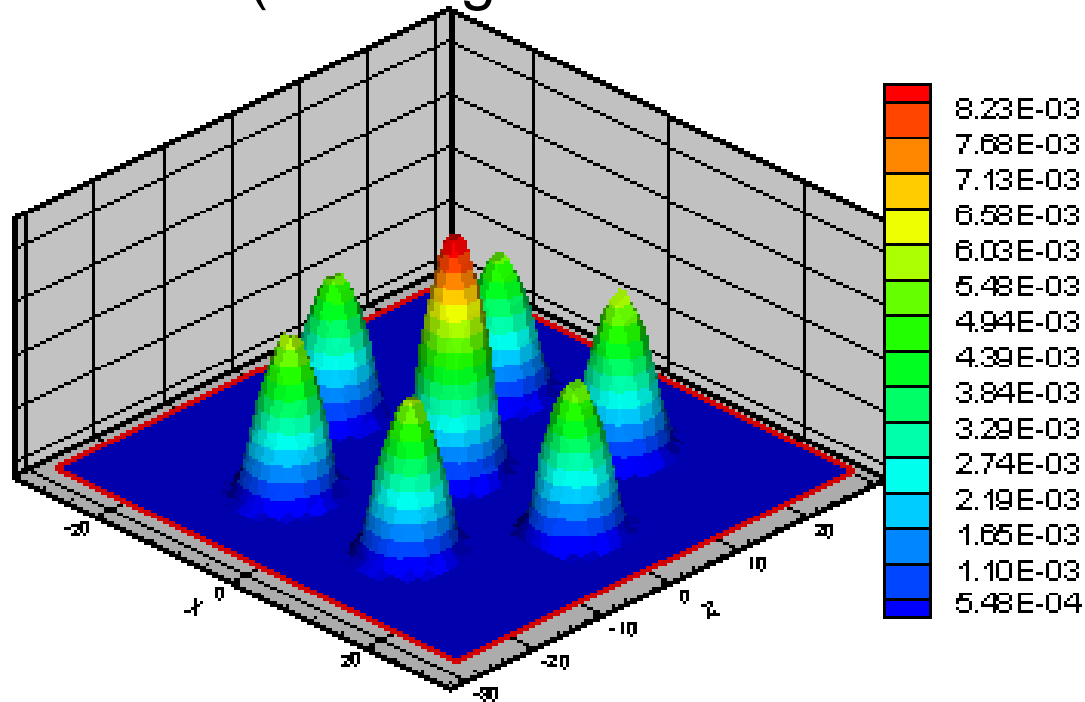
- 1,000 particles/cycle
- 100 active cycles (30 settle)
- Factor 3-4 flux tilt

- 100,000 particles/cycle
- 100 active cycles (30 settle)
- ~10% flux tilt



# Eigenfunction from the New Method

- 1,000 particles/cycle
- 100 active cycles (30 settle)
- ~10% flux tilt (converges ~100 faster than SM)



# Spherical Weight Windows

Disk of 10 MeV photons into 95cm H2O surrounding 3cm HEU

```
1 1 -19.0 -1 imp:p=1
2 2 -1.0 +1 -2 imp:p=1
3 0 +2 -3 imp:p=1
4 0 -3 imp:p=0
```

```
1 sph 0 0 0 3
2 sph 0 0 0 100
3 sph 0 0 0 200
```

```
mode p
sdef erg=10 pos -105 0 0 rad=d1 axs=1 0 0 ext=0
vec=1 0 0 dir=d2
```

```
si1 0 10
sp1 -21 1
si2 0 1
sp2 0 1
m1 92235 .5 92238 .5
m2 1001 2 8016 1
nps 100000
```

```
f4:p 1
```

```
wwg 4 0
```

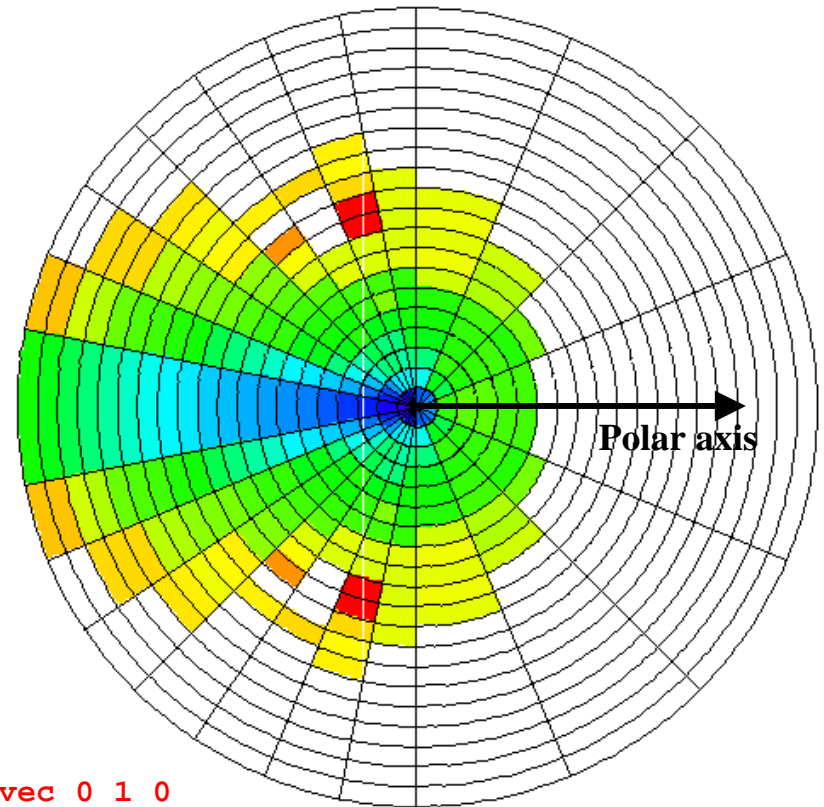
```
mesh geom rpt origin=0 0 0 ref=-99 1 1 axs 1 0 0 vec 0 1 0
```

```
imesh 101. iints 20
```

```
jmesh .25 .5 jints 4 8
```

```
kmesh 1 kints 1
```

```
c wwp:p 4j -1 $ Add this card to use WW
```



# Delayed Neutrons and Gammas

Delayed gammas from Watt fission in U-235

```
1 1 -18.9 -1 IMP:N=1
2 0 1 IMP:N=0
```

```
1 so 0.01975
```

```
mode n p
```

```
m1 92235.60c 1.
```

```
nps 20000000
```

```
sdef erg=d1 par=n
```

```
sp1 -3
```

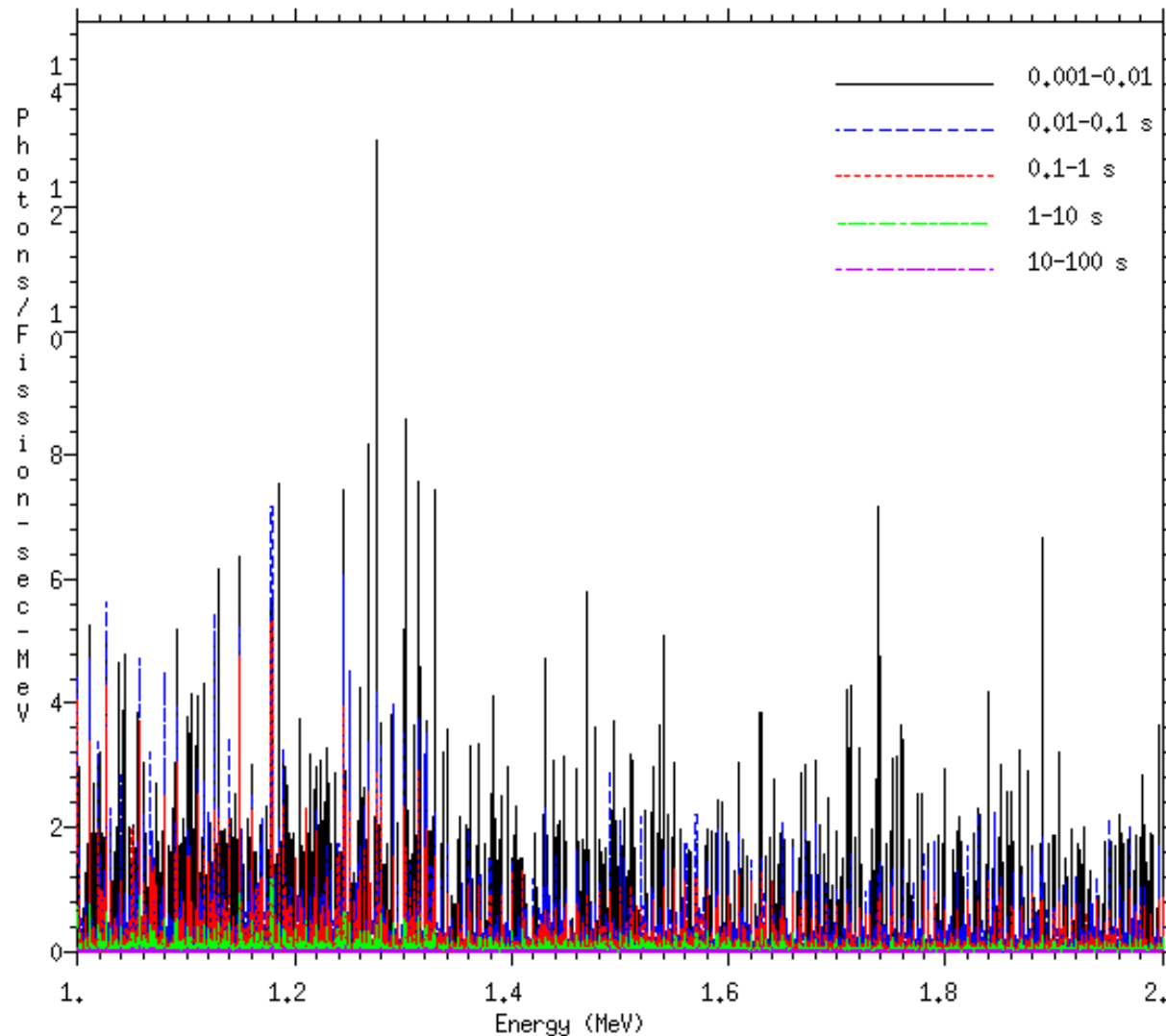
```
phys:p 5j -1
```

```
e0 0. 1024i 10.0
```

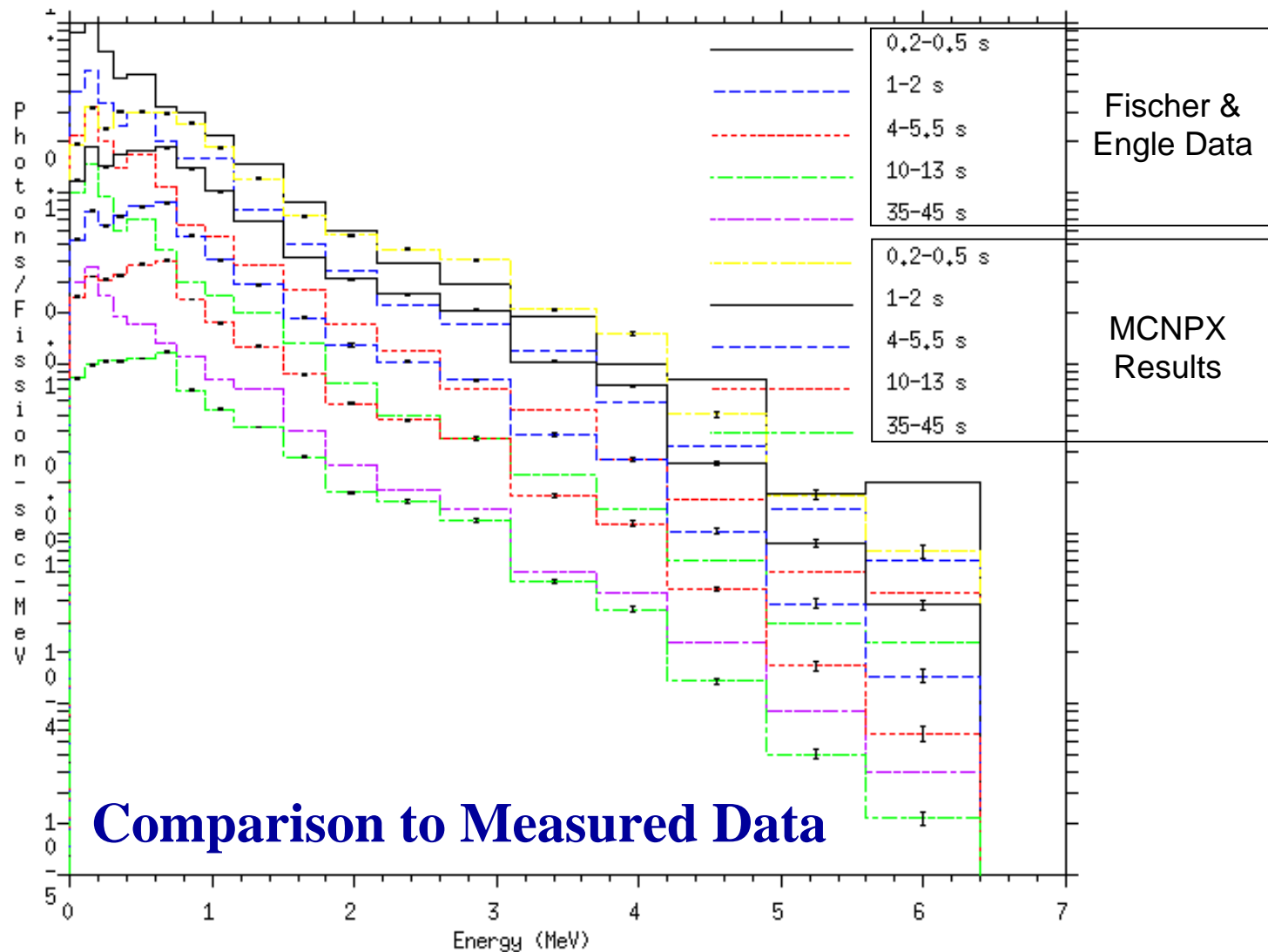
```
t0 .001e8 .01e8 .1e8 1e8 10e8 100e8
```

```
f1:p 1
```

# Delayed Neutrons and Gammas



# Delayed Neutrons and Gammas



# Photon Tally Tagging

5 GeV protons into Martian soil with photon tagging

```
1      1 -1.0      -1      imp:n=1
100    2 -1.35e-5  -101 +1   imp:n=1
101    2 -1.28e-5  -102 +101 imp:n=1
102    2 -1.22e-5  -103 +102 imp:n=1
103    2 -1.14e-5  -104 +103 imp:n=1
104    2 -1.08e-5  -105 +104 imp:n=1
105    2 -1.01e-5  -106 +105 imp:n=1
999    0          +106      imp:n=0
```

```
1      so 339000000.0
101    so 339060000.0
102    so 339110000.0
103    so 339180000.0
104    so 339240000.0
105    so 339310000.0
106    so 339380000.0
```

```
m1     8016.60c -0.6 14000.60c -0.3 26056.60c -0.1
m2     6000.60c -0.27 7014.60c -0.02 8016.60c -0.70 18000.35c -0.01
```

```
FIELD GCUT=0.1320 GPAR=1 GRAD=3393.0 GSUR=106
```

```
mode h n p z / d t s a
```

```
lca 8j 1 $ Use CEM
```

```
sdef par=9 erg=5000 sur=106 nrm=-1
```

```
nps 10000
```

```
print
```

```
phys:n 5010 j j j 20
```

```
e11 0. 1024i 10. 5000.
```

```
full1 0. 8016.00051 8016.00052 8016.00053 8016.00102 8016.
14028.14027 14028.14026 14028.13027 14028.13026 14000.
26056.00051 26056.00052 26056.00053 26056.00102 26056.
```

```
f11:p 1
```

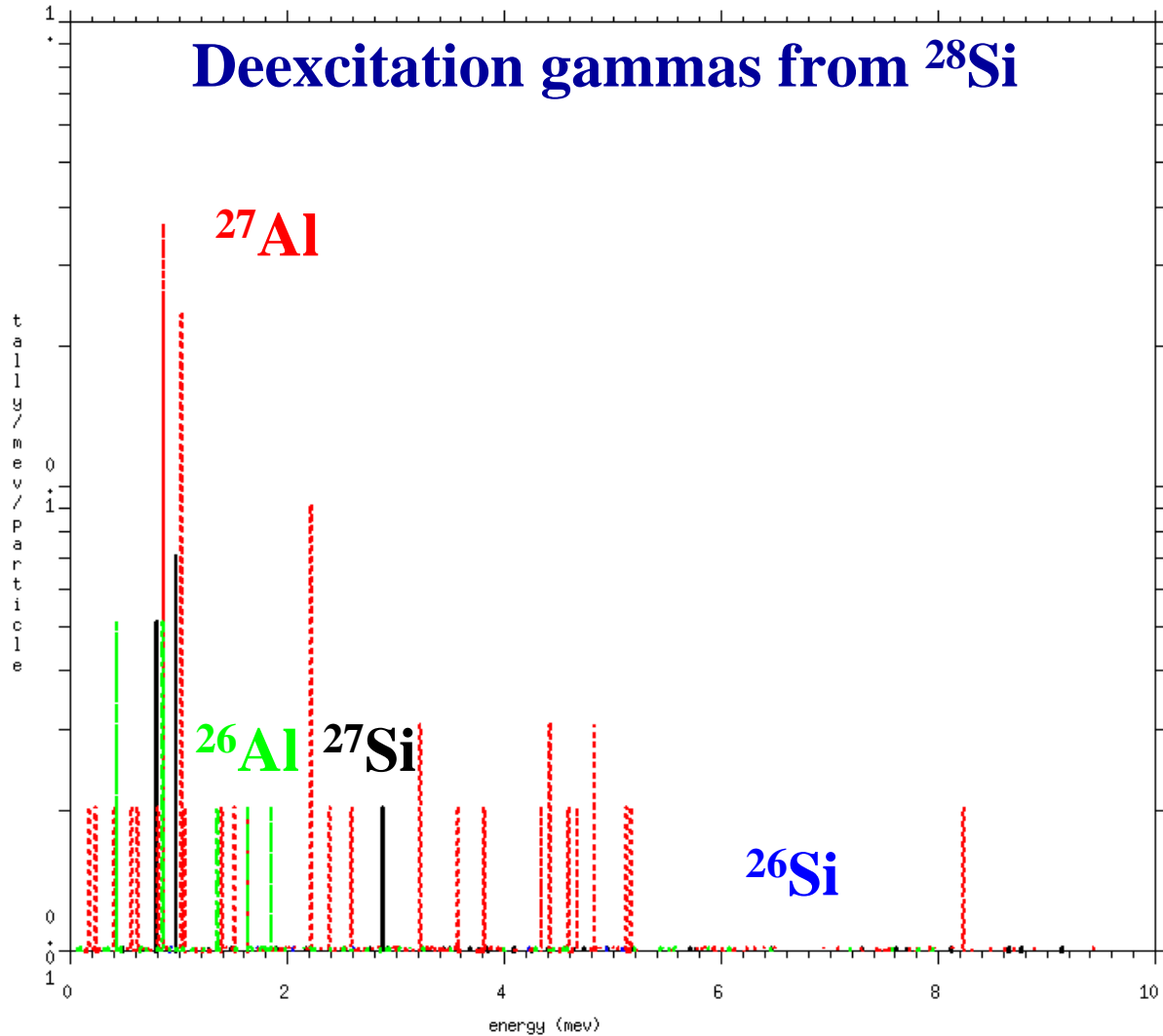
```
ft11 tag 1
```

```
e21 1e-10 99log 1e-7
```

```
f21:n 105
```

Undocumented Feature

# Photon Tally Tagging





# Models for Library Interactions

## 2 MeV neutrons into He-3

```
1  1 -5.3540E-4  -1  imp:n=1
2  0                1 -2 imp:n=1
3  0                2   imp:n=0
```

```
1  so 4.0
2  so 100.0
```

```
mode  n h d t s
sdef  par=n erg=2 pos=-5 0 0 rad=d1
      axs=1 0 0 ext=0 vec=1 0 0 dir=1
```

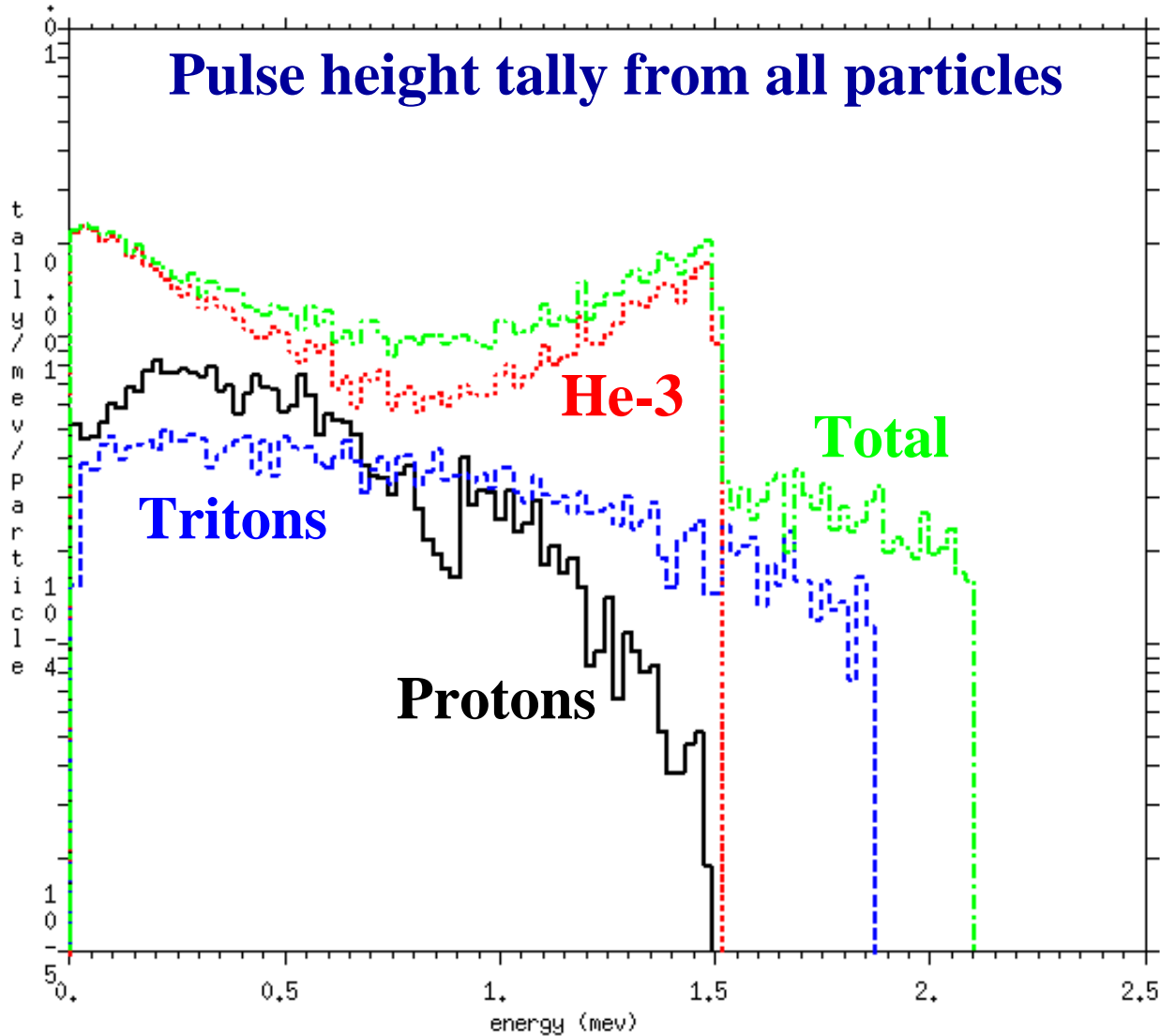
```
sil   0 3
spl   -21 1
cut:n  2j 0 0
cut:h,d,t,s j .001
```

## phys:n 6j 2

```
m1    2003.60c 1
nps   10000000
f6:h  1
f16:d 1
f26:t 1
f36:s 1
```

```
f8:n  1
e8    0. 99i 2.1
ft8   PHL 1 6 1 0
f18:n 1
e18   0. 99i 2.1
ft18  PHL 1 16 1 0
f28:n 1
e28   0. 99i 2.1
ft28  PHL 1 26 1 0
f38:n 1
e38   0. 99i 2.1
ft38  PHL 1 36 1 0
f58:n 1
e58   0. 99i 2.1
ft58  PHL 4 6 1 16 1 26 1 36 1 0
```

# Models for Library Interactions



# Future of MCNPX

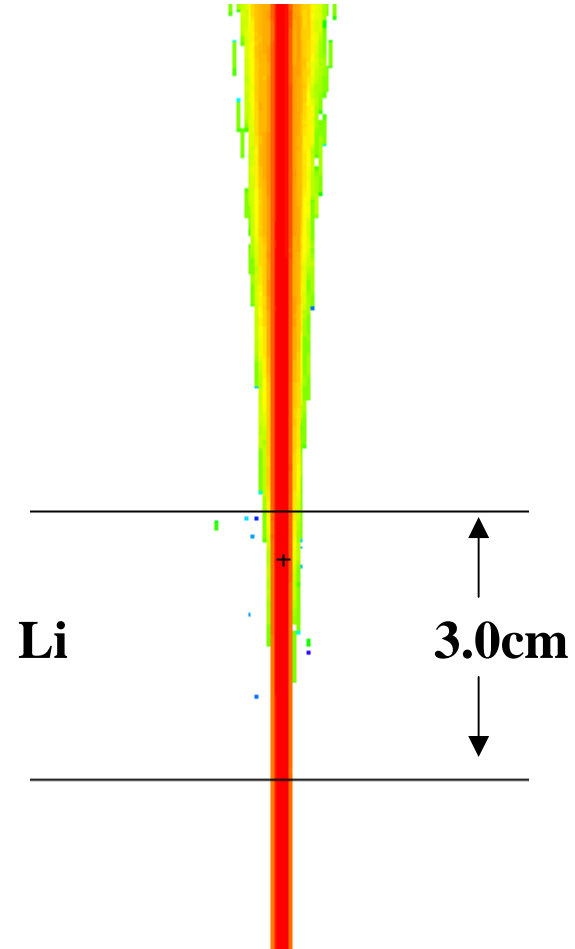
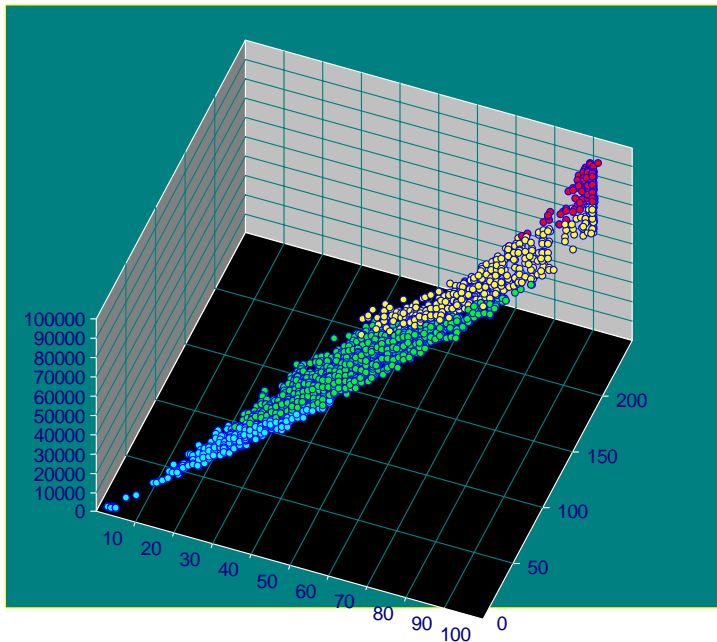
- **Possible public release of 2.6.0 (Summer 2007)**
- **Version 2.6.D/2.7.A (April 2007)**
  - **Transmutation improvements**
  - **Energy and time weight windows**
  - **Radioactive source option**
  - **Photofission and delayed neutron improvements**
- **MCNPX and MCNP merger**
  - **Hope to preserve all features of both codes**
  - **Preliminary version by Summer 2007**
  - **Public release perhaps by 2008**

# Future Development

- **Heavy-ion tracking and LAQGSM physics model (2.6.?)**
- **Magnetic fields**
- **CAD interface (with spline-surface tracking)**
- **Variance reduction techniques extended to models**
- **Improve point detectors/DXTRAN for models**
- **Extend electron data to 100 GeV**

# Heavy Ion Transport

- Example: U-238 at 400 GeV/nucleon traveling through a Li jet.



# Existing Burnup Capabilities

- **Numerous “scripts” written to link MC codes to depletion codes**
  - MOCUP (MCNP/ORIGEN2, INEL, 1995)
  - MC-REBUS (MCNP/REBUS, ANL, 1998)
  - OCTOPUS (MCNP/FISPACT, ECN NRG Netherlands, 1998)
  - MCB (MCNP/Custom, RIT Sweden, 1999)
  - MonteBurns2 (MCNP/ORIGEN2 or CINDER90, LANL, 1999)
  - MCWO (MCNP/ORIGEN2, INEEL, 2000)
  - BURNCAL (MCNP/Custom, SNL, 2002)
  - MCODE (MCNP/ORIGEN2, MIT, 2002)
- **Disadvantages of a “link” approach**
  - Several input files to create and understand
  - Numerous input/output files to manage
  - Approximations to convert data from one format/code to another

# MCNPX/CINDER90 Interface

- **MCNPX provides to CINDER90**
  - 63-group fluxes in each material to be burned
  - Isotopic atom densities and material volumes
  - Absorption and fission reaction rates for each nuclide
  - Average  $k_{\text{eff}}$  and fission  $\nu$ , and fission  $Q$
  - Power level and burn time
- **CINDER90 provides to MCNPX**
  - Updated isotopic atom densities
  - Burnup quantities
- **User interface (BURN card)**
  - BURN card without any entries defaults to 1 MW power for 1 day
  - User can specify burn materials, power level, burn times, etc.
  - Histories run per burn time are taken from NPS or KCODE card

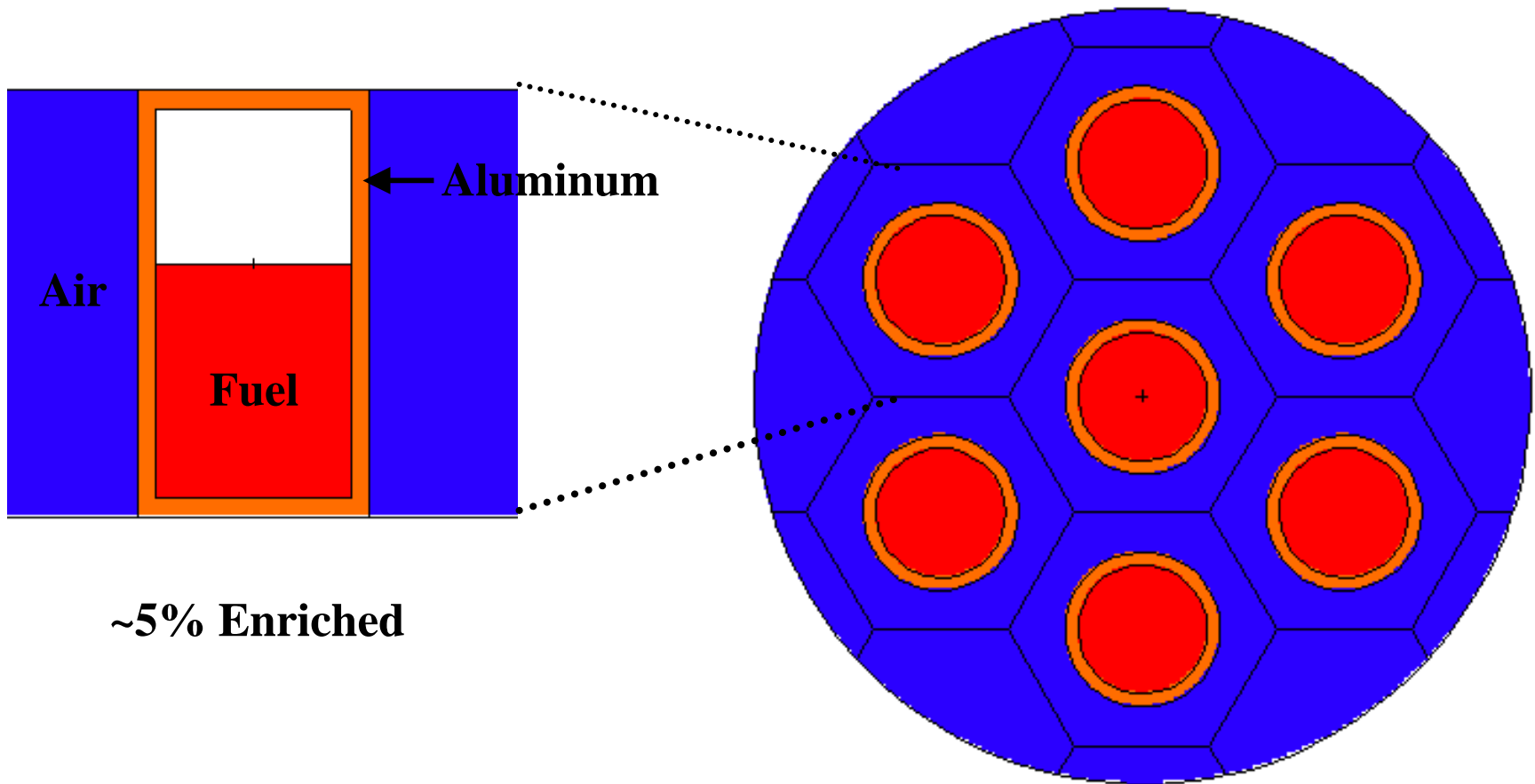
# BURN Card

BURN POWER=2.0 TIME=15,30,30 MAT=3,4  
OMIT=3,3,8017,92234,94239,4,1,92234

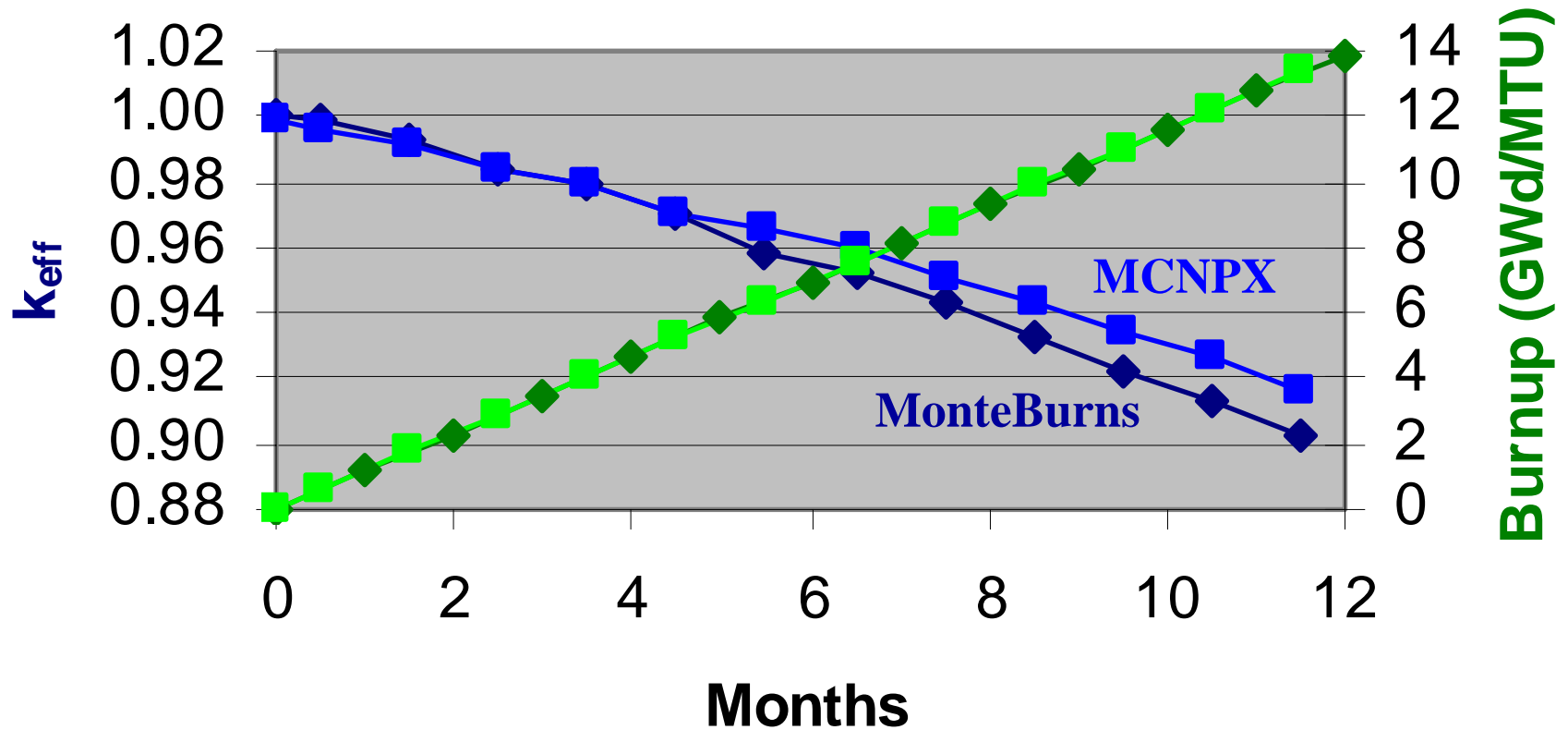
Specifies a power level of 2 MW for a duration of 75 days (steps of 15, 30, and 30 days). Materials 3 and 4 are included in the burn with isotopes  $^{17}\text{O}$ ,  $^{234}\text{U}$ , and  $^{239}\text{Pu}$  excluded from material 3 and isotope  $^{234}\text{U}$  excluded from material 4. Nuclides with an atom fraction less than  $1\text{e-}10$  are also excluded. To force the inclusion of a nuclide, simply list that nuclide on the appropriate material card with an insignificant atom fraction.



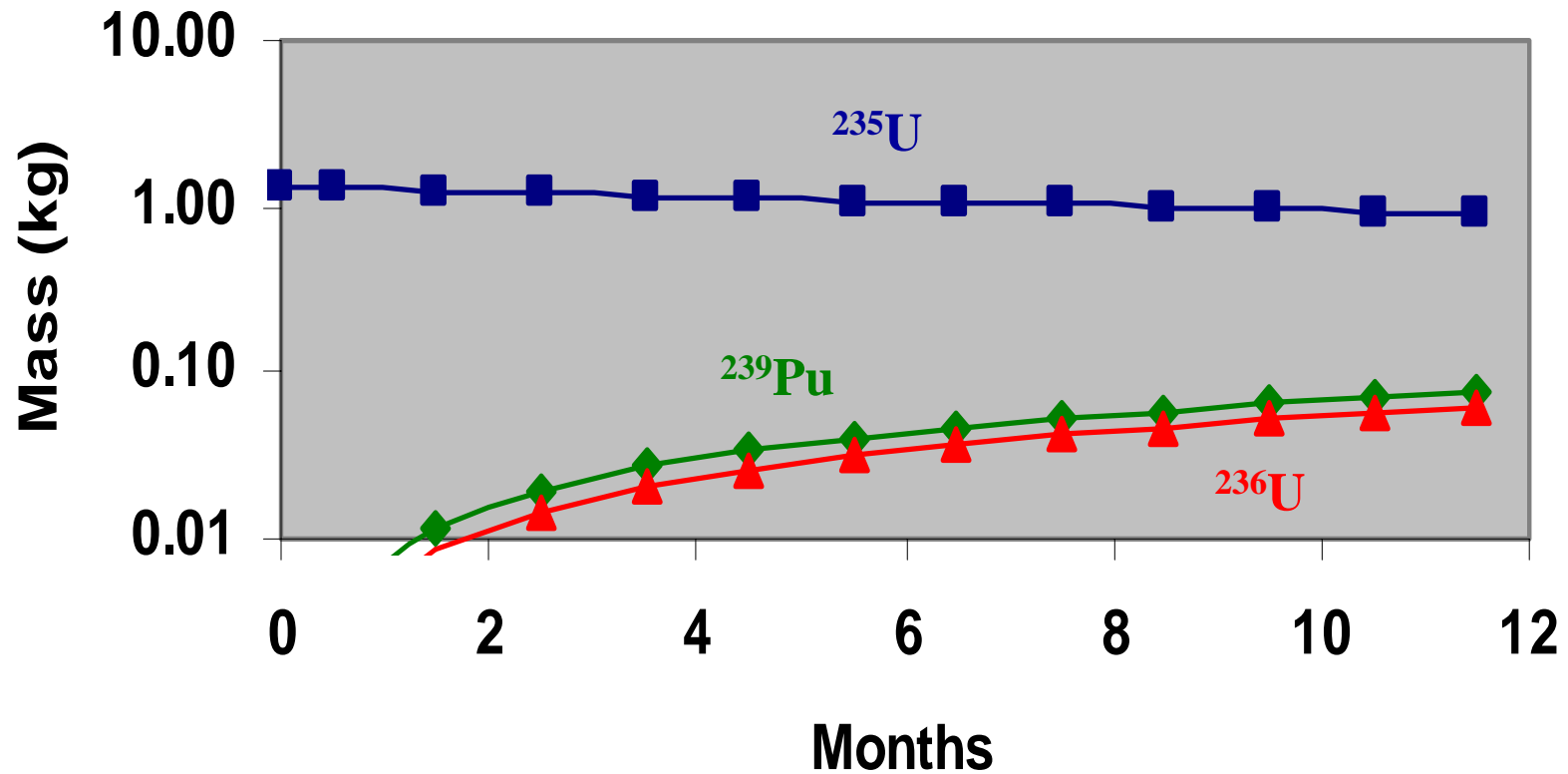
# 7-Can HEU Test Problem



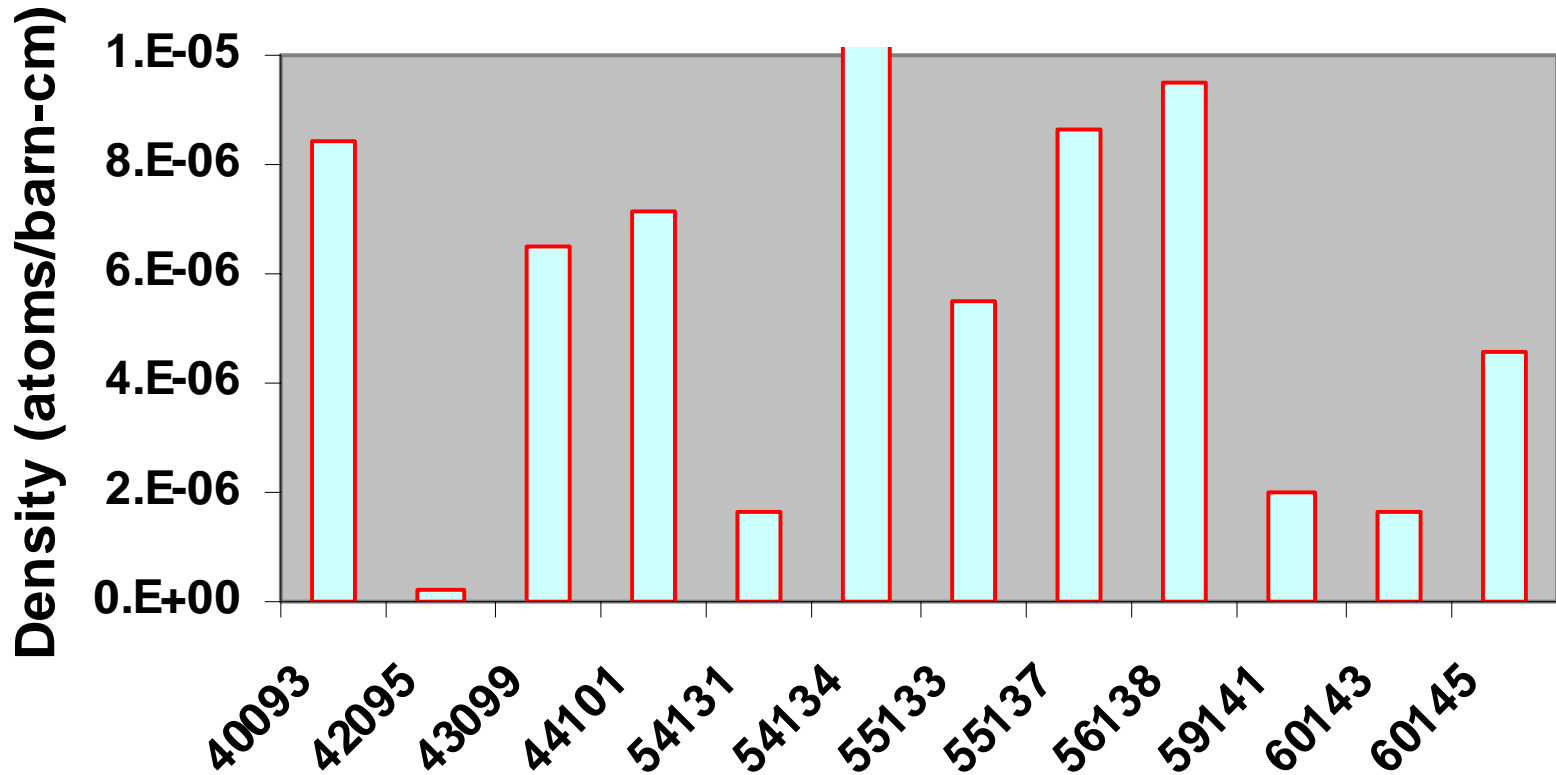
# Comparison to MonteBurns



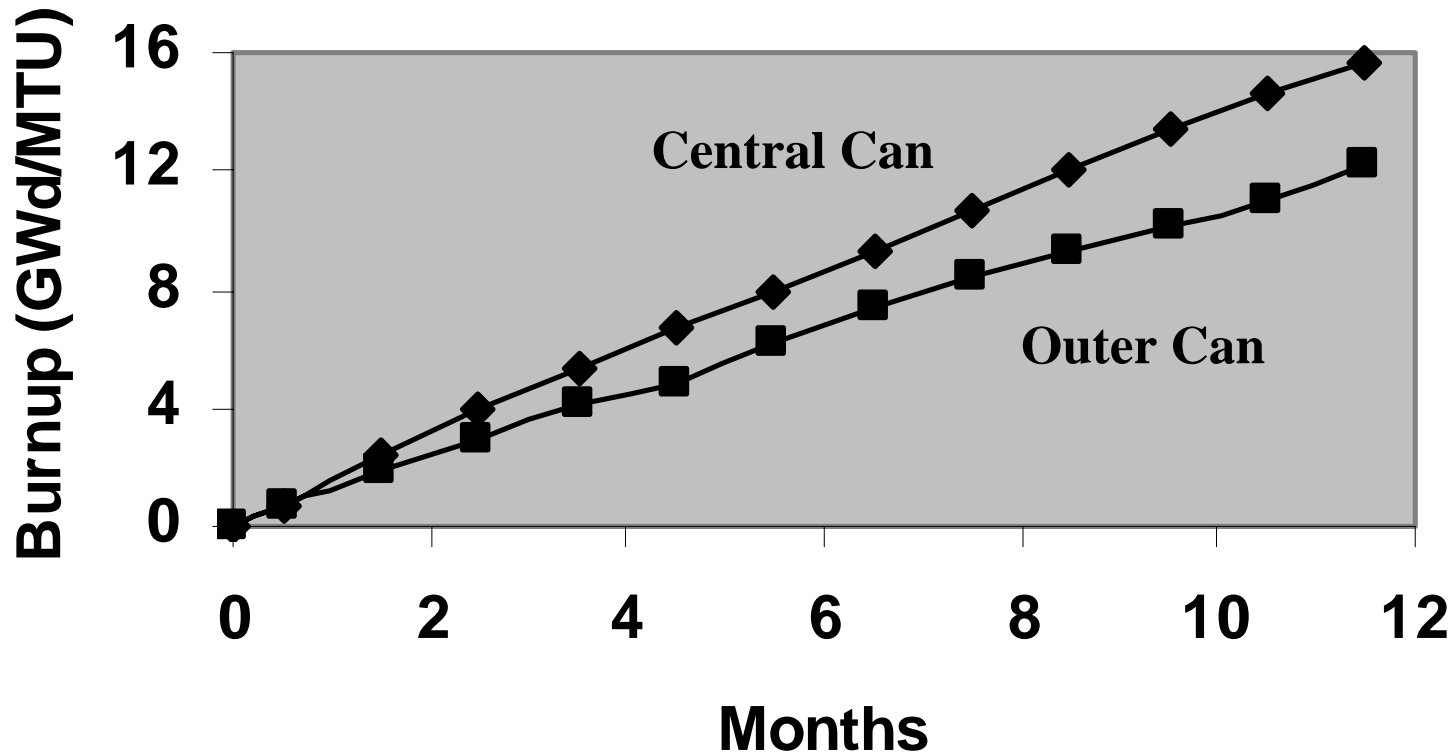
# Actinide Inventories



# Fission Product Inventories



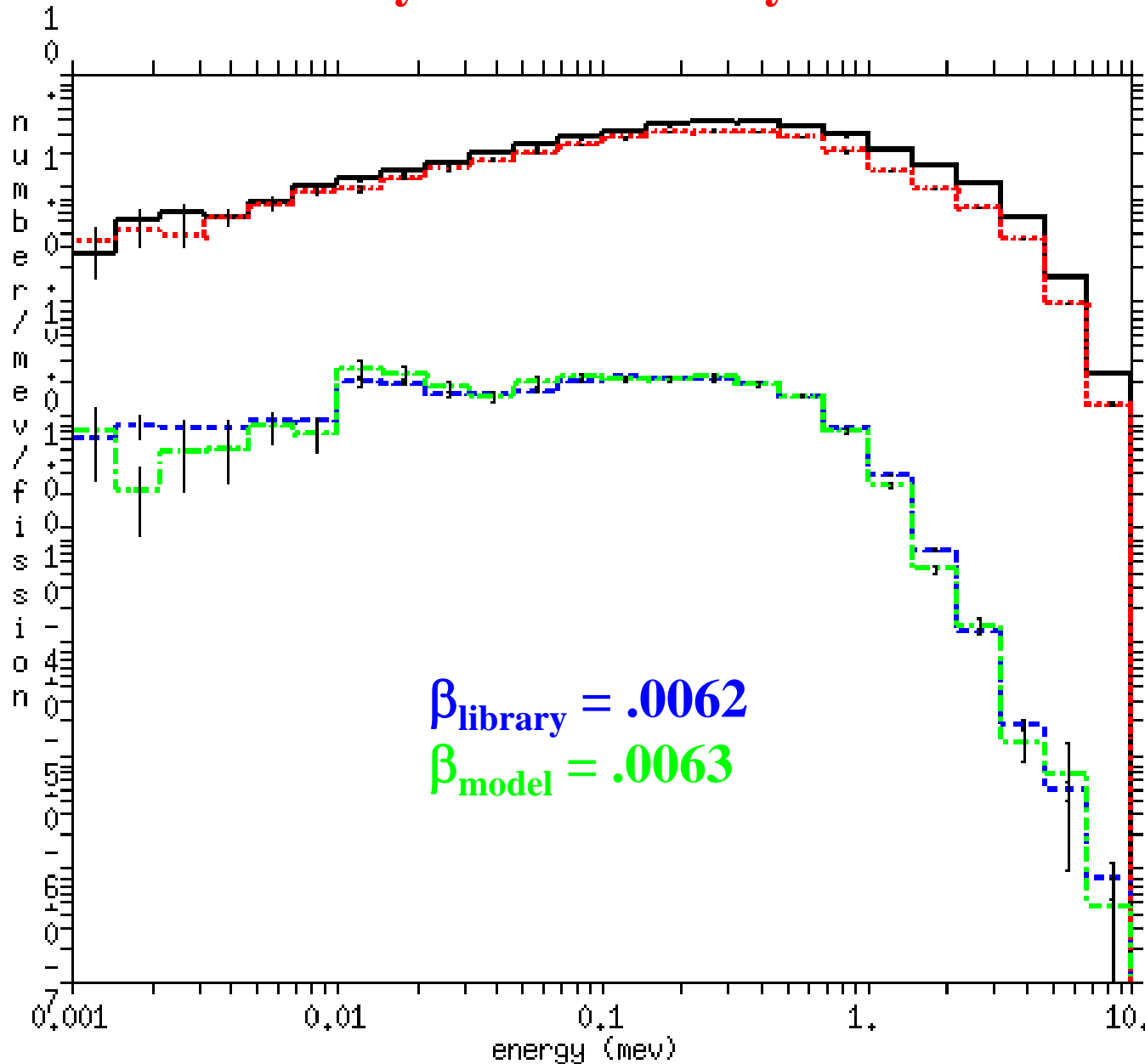
# Can Burnup



# Delayed Neutron/Gamma Capability

- Delayed treatment for model and library interactions
  - All events for model interactions
  - Fission events for library interactions
- Delayed distributions provided by CINDER90
  - Uses residual nuclides from model interactions
  - Samples fission products for library fission events
  - Includes virtually all possible decay chains
  - Provides energy distributions in groups (300 n, 25 g)
  - Provides time distributions (continuous => 70 bins)
- Biasing available to enhance delayed production

# <sup>235</sup>U Library vs Model Delayed Neutron Energy Spectra

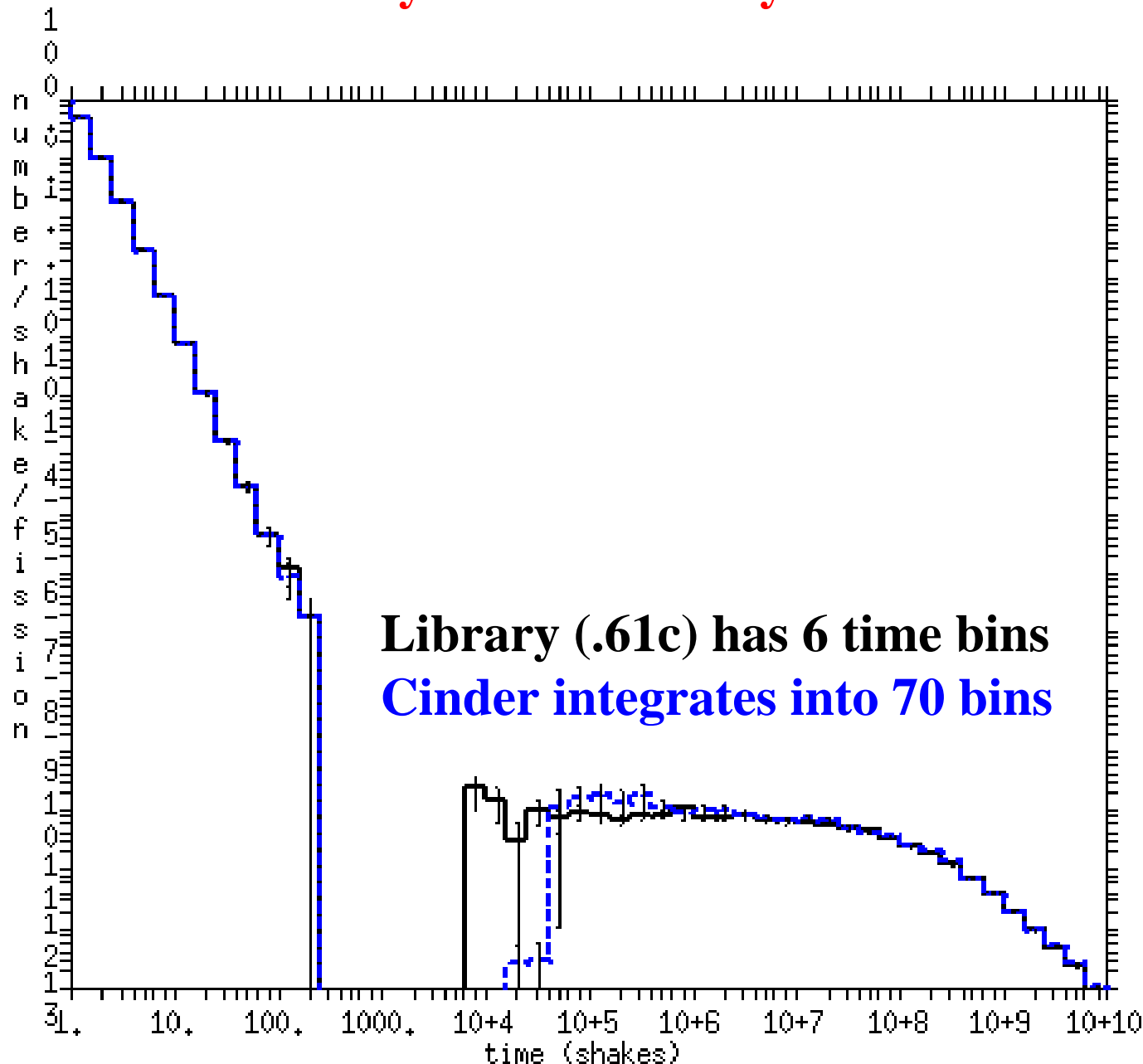


```

mcnpx          25e42
               05/13/04 13:46:58
tally         1
n
nps          10000000
bin normed
runtp = d000b,r
dump         6
f  surface    1
d  flag/dir   1
u  user       2
s  segment    1
m  mult       1
c  cosine     1
e  energy     *
t  time       1
    
```

— prompt old  
 - - - delay .61c  
 . . . prompt new  
 - . - . delay CIND

# $^{235}\text{U}$ Library vs Model Delayed Neutron Time Spectra

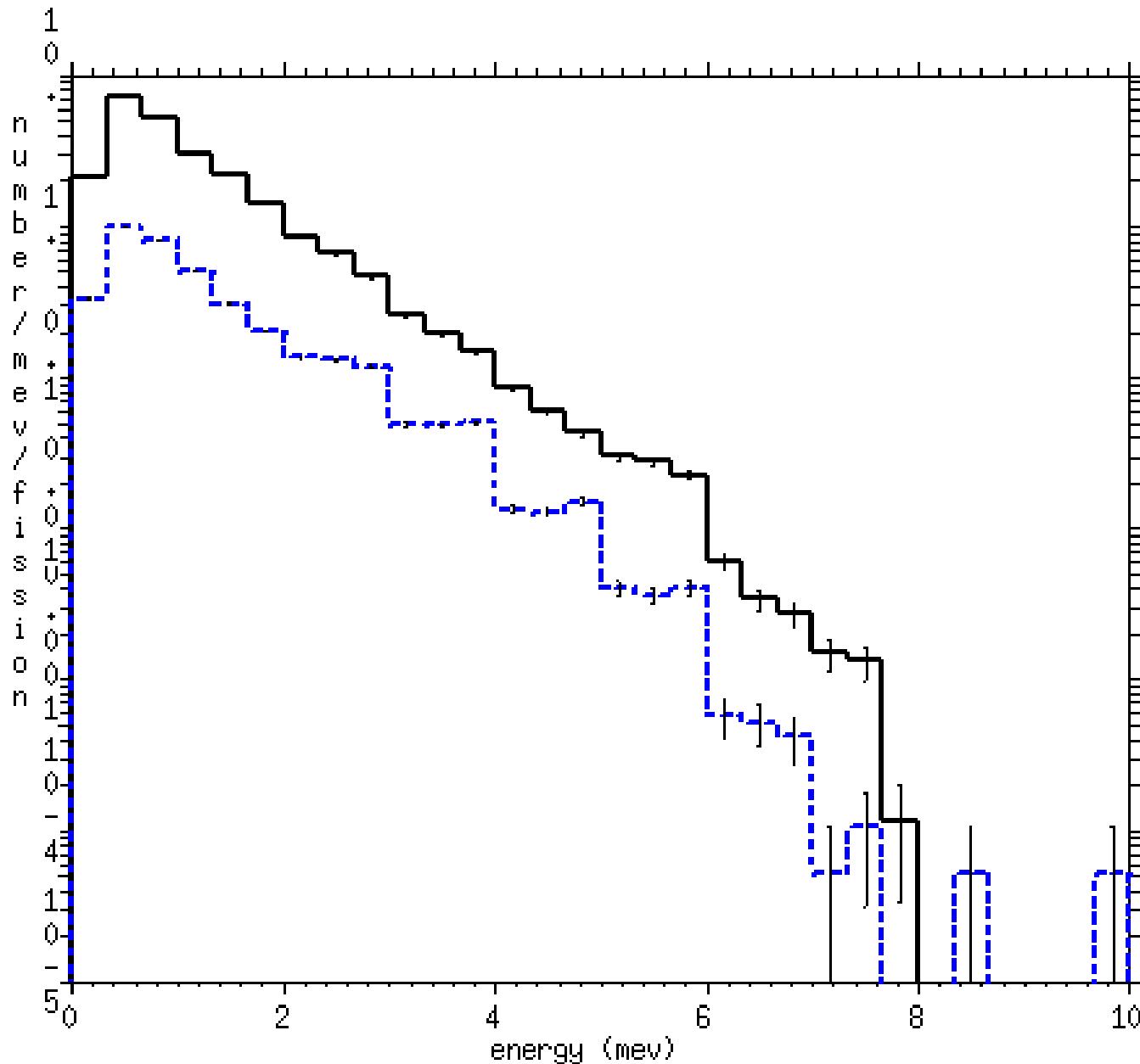


```
mcnpX      25e42
           05/12/04 14:27:33
tally     11
n
nps       10000000
bin normed
runtp = d000b,r
dump      6
f surface 1
d flag/dir 1
u user    1
s segment 1
m mult    1
c cosine  1
e energy  1
t time    *
```

— .61c  
- - - Cinder



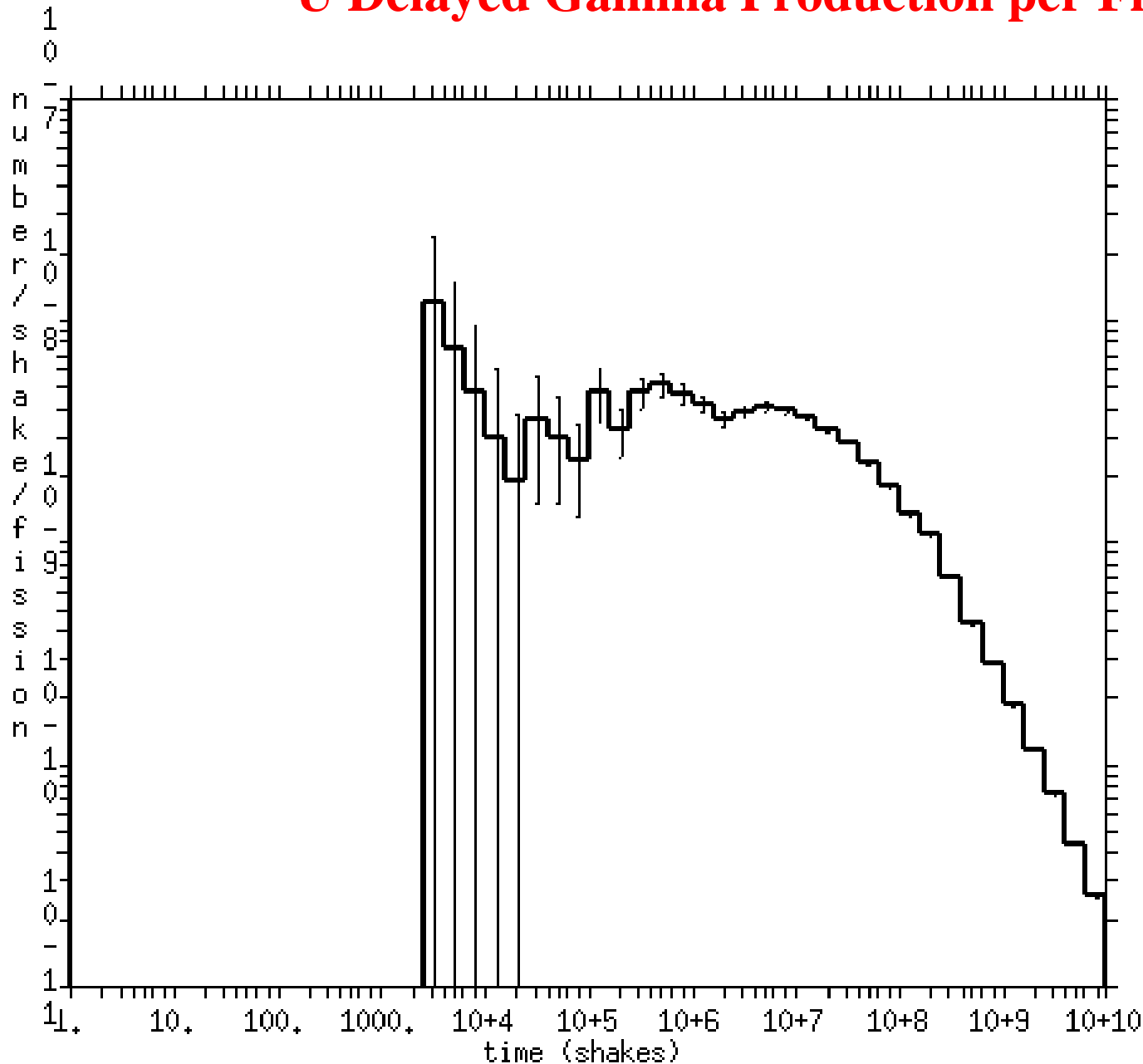
# $^{235}\text{U}$ Delayed Gamma Production per Fission



```
mcnpX          25e42
  05/12/04 14:15:42
tally         21
P
nps          10000000
bin normed
runtpe = d000a,r
dump          6
f  surface    1
d  flag/dir   1
u  user       1
s  segment    1
m  mult       1
c  cosine     1
e  energy     *
t  time       1
```

— prompt  
- - - delay

# $^{235}\text{U}$ Delayed Gamma Production per Fission



```
mcnpX          25e42
  05/12/04 14:15:42
tally    31
P
nps      10000000
bin normed
runtpe = d000a.r
dump     6
f  surface    1
d  flag/dir   1
u  user       1
s  segment    1
m  mult       1
c  cosine     1
e  energy     1
t  time       *
_____ d000a.r
```